

KFKI-1986-40/G

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MEASUREMENT OF REACTIVITY TEMPERATURE  
COEFFICIENT BY NOISE METHOD  
IN A POWER REACTOR

*Hungarian Academy of Sciences*

**CENTRAL  
RESEARCH  
INSTITUTE FOR  
PHYSICS**

**BUDAPEST**

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PREPRINT

MEASUREMENT OF REACTIVITY TEMPERATURE  
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IN A POWER REACTOR

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HU ISSN 0368 5330

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## ABSTRACT

The temperature reactivity coefficient has been estimated on the basis of noise measurements performed in a PWR. The magnitude of the coefficient has been evaluated by relating the values of the APSD and CPSD between ex-core neutron detector signals and fuel assembly outlet thermocouple in the low frequency range. Comparison with  $\partial\rho/\partial T$  measurements performed in PWR by standard method supports the validity of the results.

## АННОТАЦИЯ

Температурный коэффициент реактивности был измерен в реакторе ВВЭР-440 новым шумовым методом. Для определения данного параметра использовались функции спектральной мощности взаимной корреляции и автоспектральной мощности ионизационной камеры и термопар в области низких частот. Сравнение измеренных значений  $\partial\rho/\partial T$  со значениями, полученными по стандартной методике, показывает хорошее согласие.

## KIVONAT

VVER típusu atomerőműben végzett zajmérések alapján meghatároztuk a reaktivitás hőmérséklet együtthatóját. Erre a célra a zónán kívüli ionizációs kamerák és a zóna kilépésénél elhelyezett termoelemek CPSD és APSD függvényeinek alacsony frekvenciás részét használtuk fel. A standard módszerekkel nyert eredményekkel történt összevetés alátámasztja az új módszer használhatóságát.

## 1. INTRODUCTION

The feasibility of using neutron flux and core-exit temperature signals in power reactors to estimate the reactivity temperature coefficient has been demonstrated in previous works [1,2,3].

Theoretical studies performed in the references mentioned suggested that, on the basis of noise measurement, the reactivity temperature coefficient in PWR can be estimated using the following expression:

$$\partial\rho/\partial T = G(\omega) \cdot \frac{\text{CPSD}(\omega)}{\text{APSD}(\omega)} \quad (1)$$

where:  $\text{CPSD}_{N,T}(\omega)$ : Cross Power Spectral Density between neutronic and temperature signals.

$\text{APSD}_T(\omega)$  : Power Spectral Density for temperature sensor.

$G(\omega)$  : Reactor Transfer function; for PWE-WWER  
 $G(\omega)^{-1} \sim \beta_{\text{EFF}} = 0.0075$

This paper is concerned with the evaluation of  $\partial\rho/\partial T$  coefficient using expression (1) applied to noise measurements performed in Paks NPP Unit 2.

## 2. RESULT OF MEASUREMENTS AND DISCUSSION

To support the validity of expression (1) noise signals recorded in Paks NPP Unit 2 during the second fuel cycle, under normal operation, were analyzed and evaluated. Unit 2 is equipped with a sophisticated noise measuring system consisting of in-core neutron detectors, ex-core ionization chambers and fuel assembly

outlet thermocouples [4]. The positions of the ex-core, in-core neutron detectors and in-core fuel assembly outlet thermocouples are shown in *Fig. 1*.

A set of four ex-core detectors was used to estimate the  $\partial\rho/\partial T$  coefficient for the Paks NPP Unit 2. *Fig. 2* shows the global character of the signals used in the evaluation.

The signals are connected to noise preamplifiers positioned near to the reactor vessel. The preamplified noise signals are cabled to a noise diagnostic laboratory near to the control room of the unit, where the fluctuating signals are further amplified and recorded by a 14-channel FM tape recorder.

All the signals were normalized considering the different amplification factors (internal amplification factor of Paks amplifiers, external amplification factor, etc.). For the thermocouples a factor of  $4 \times 10^{-5}$  volt/ $^{\circ}\text{C}$  was used, and an amplification factor of  $10^5$  was considered.

The neutron noise signals from each detector were low pass filtered at 4 Hz to prevent aliasing, high pass filtered at 0.01 Hz, digitized with a sampling interval of 0.125 sec, partitioned into blocks of 256 points, Hanning windowed and Fourier transformed. APSDs and CPSDs between pairs of signals were analyzed.

Table 1 shows the magnitude of the coefficient  $\partial\rho/\partial T$  estimated for different detector-thermocouple set. The coherence was found in all the cases over 10% and the effect of the temperature fluctuation on the neutron spectrum was local between 0.3-0.4 Hz.

In table 1 it can be noticed the values calculated from the CPSD and APSD pictures at the set of points where the effect was localized.

The last row of table 1, shows the mean value estimated from the set of measurements and the calculated statistical error. These results indicate that the inferred error does not reach 10%.

In *Figs. 3,4* a typical phase behaviour between an ex-core detector and a thermocouple can be observed (see *Fig. 1* for details of detector positions). In the low frequency range, one can observe that a peak appears at 0.63 Hz. We suppose that this peak belongs to the pressure fluctuation in the coolant that affects the neutron spectrum. These results were obtained for this reactor reported in previous works [5,6].

To support the validity of expression (1), the results showed in Table 1, were compared with results obtained by standard forced dynamics methods used in PWRs of WWER type.

In Table 2 the results reported in different papers for measurements performed in PWR-WWER type were summarized.

Since in the references mentioned the measurements were performed in somewhat different conditions, some interpolation of the results should be made. Nevertheless, the results obtained with our method, agree well qualitatively and quantitatively with those reported by other authors, and this support the validity of the method.

## 2. CONCLUSION

Considering the APSD and the CPSD between a neutronic detector and a thermocouple signal, the value of the temperature coefficient has been estimated for a set of different combinations of ex-core detectors and outlet assembly thermocouples in an NPP of PWR.

The comparison of these results obtained for the temperature coefficient with measurements performed by standard forced dynamics methods in PWR core, supports the validity of the model emphasized and the results obtained.

## ACKNOWLEDGEMENTS

The autor thanks Dr. G. Pór for his continuous contribution to this paper and Dr. L. Meskó for correcting the manuscript.

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Table 1

Measurement detectors	peak freq. (Hz)	coherence (%)	NCPSD <sub>N,T</sub> (°C/Hz)	NAPSD <sub>T</sub> (°C <sup>2</sup> /Hz)	$\partial\rho/\partial T$ (°C <sup>-1</sup> )
P2B2K.2A JK-1/TH-5	0.375	12.25	1.433 10 <sup>-6</sup>	8.56 10 <sup>-5</sup>	-1.26 10 <sup>-4</sup>
P2B2K.2A JK-18/TH-5	0.375	10.69	1.259 10 <sup>-6</sup>	8.56 10 <sup>-5</sup>	-1.10 10 <sup>-4</sup>
P2B2K.2A JK-9/TH-5	0.344	13.90	1.523 10 <sup>-6</sup>	1.14 10 <sup>-4</sup>	-1.00 10 <sup>-4</sup>
P2B2K.2A JK-10/TH-5	0.344	12.65	1.456 10 <sup>-6</sup>	1.14 10 <sup>-4</sup>	-0.96 10 <sup>-4</sup>
P2B2K.2B JK-9/TH-5	0.375	13.17	1.165 10 <sup>-6</sup>	7.39 10 <sup>-5</sup>	-1.18 10 <sup>-4</sup>
P2B2K.2B JK-9/TH-5	0.375	12.75	1.400 10 <sup>-6</sup>	8.91 10 <sup>-5</sup>	-1.18 10 <sup>-4</sup>
P2B2K.2B JK-18/TH-5	0.320	18.00	2.020 10 <sup>-7</sup>	1.51 10 <sup>-5</sup>	-1.01 10 <sup>-4</sup>

Mean Value:  $(-1.10 \pm 0.10) \cdot 10^{-4} \text{ } ^\circ\text{C}^{-1}$



Table 2

Reference/ Method.	Reactor Type	Reactivity coeff. ( $10^{-4} \text{ } ^\circ\text{C}^{-1}$ )	Temperature ( $^\circ\text{C}$ )	Boron Conc. (g/Kg)
Obchinikov and co-workers Dynamics stand. methods (16)	WWER-440 Unit 4, Nowovoronezh NPPP	-2.1 -0.5	285 285	5.72 8.58
This Work/ Noise Method.	WWER-440 Unit 2, Paks NPP	-1.10	282.1	6.9
Adorjan F. and co-workers (17) *	WWER-440 Unit 1, Paks NPP	-0.1	260	8.0

(\*) At zero power.

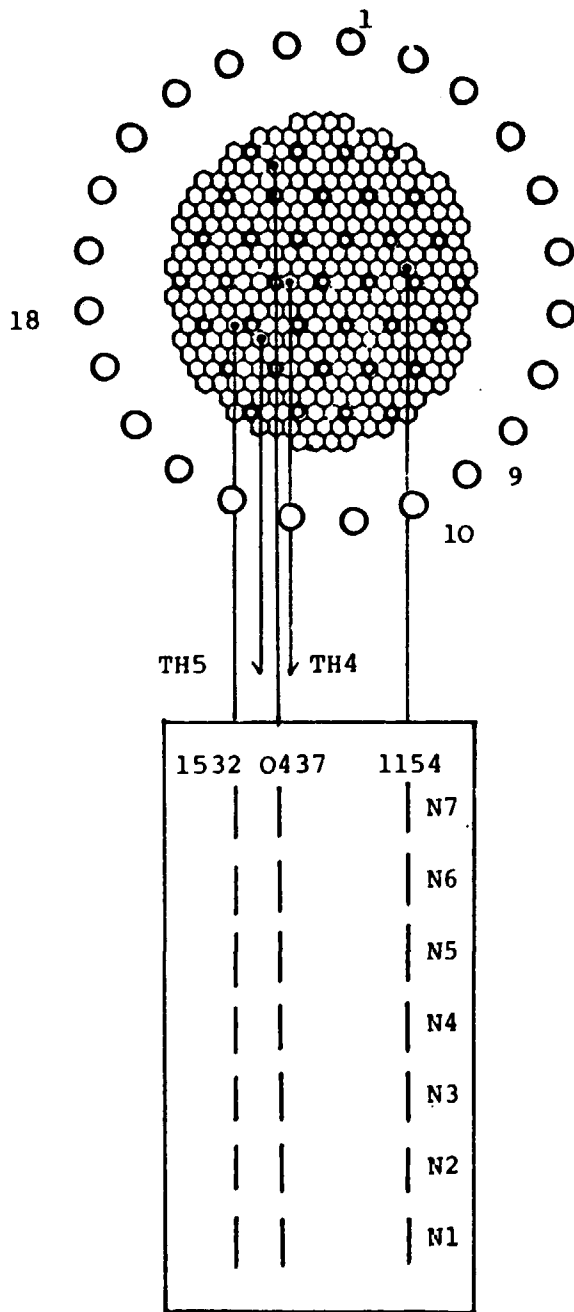


Fig. 1.

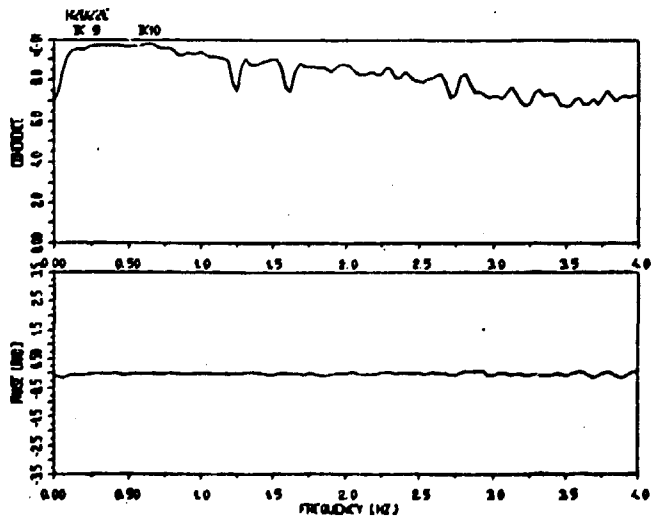
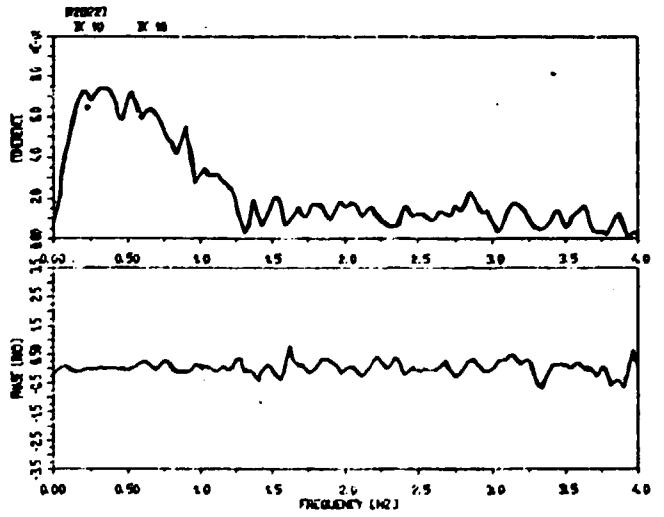
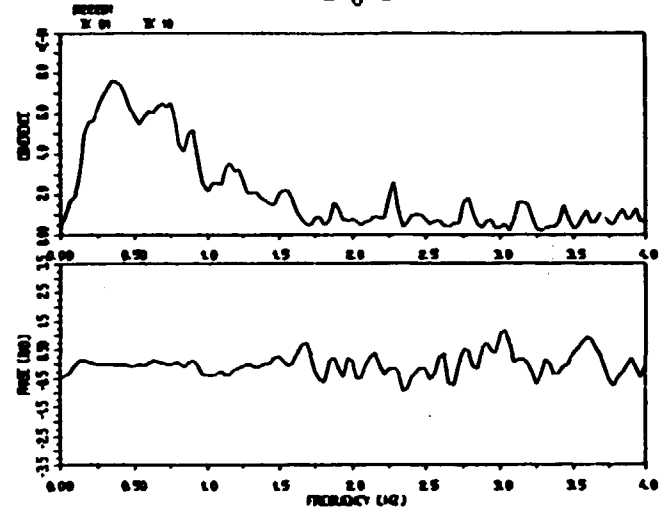


Fig. 2.

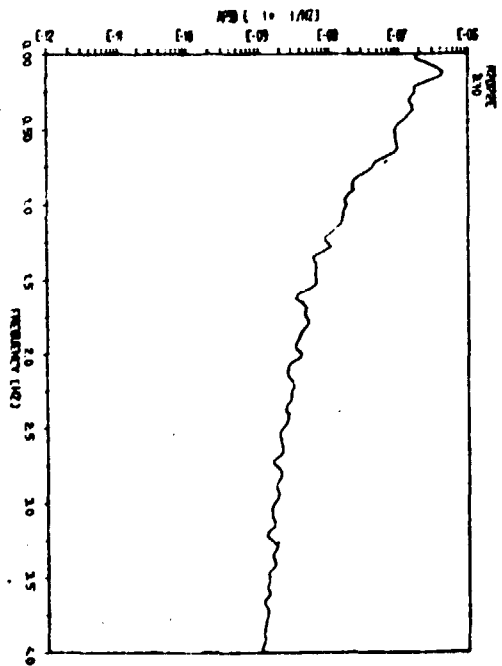
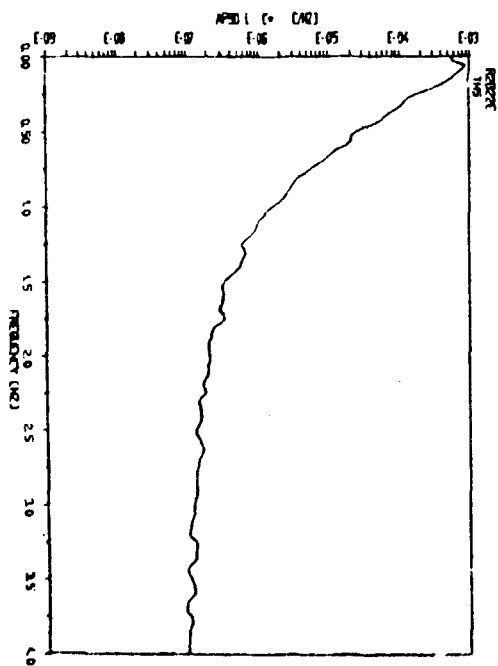
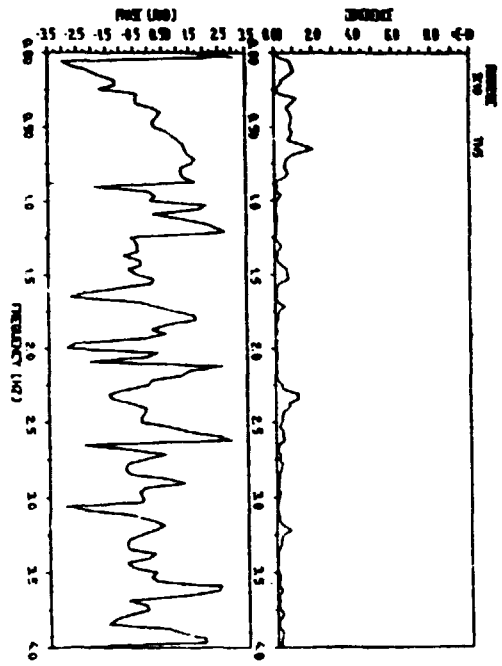
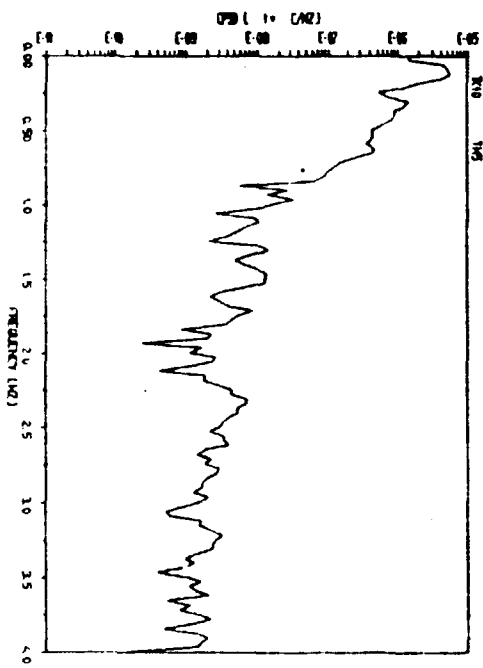


Fig. 3.

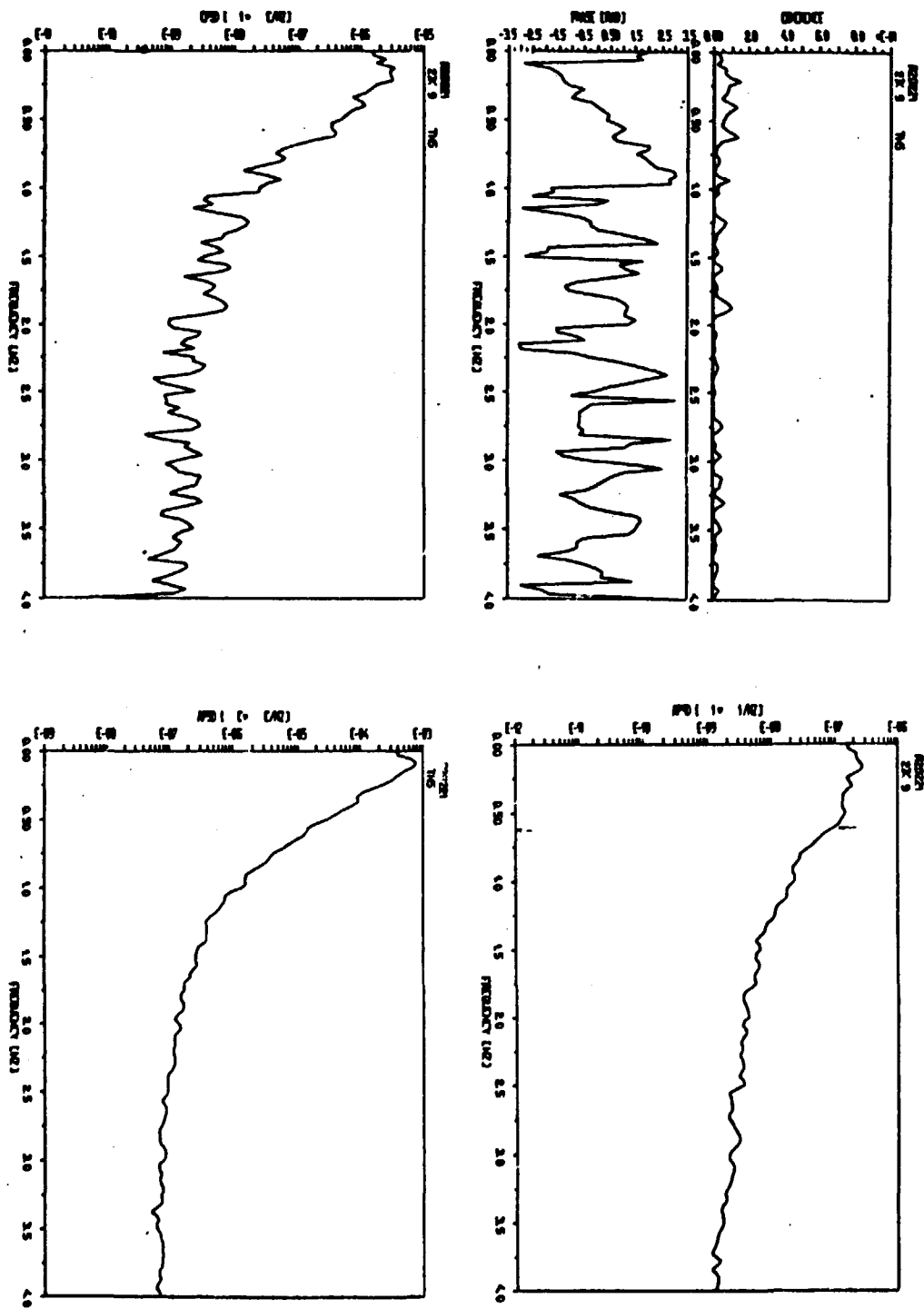


Fig. 4.

FIGURE CAPTIONS

*Fig. 1. Positionsof in-core fuel assembly outlet thermocouples and in-core, ex-core detectors.*

*Fig. 2. Coherence and phase between ex-core detectors.*

*Fig. 3. } CPSD, APSD, coherence and phase between*  
*Fig. 4. } ex-core detectors and thermocouples.*

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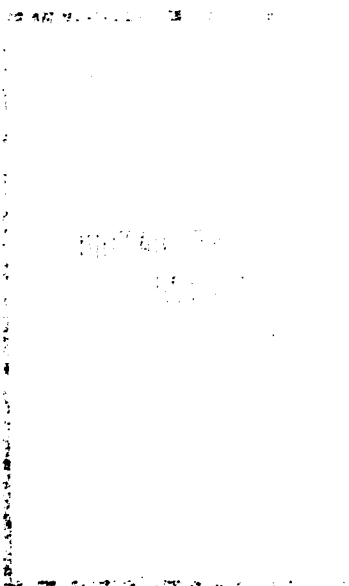
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