
Topical Report Review Status

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U.S. Nuclear Regulatory Commission
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ABSTRACT

This report provides industry with procedures for submitting topical reports, guidance on how the U.S. Nuclear Regulatory Commission (NRC) processes and responds to topical report submittals, and an accounting, with review schedules, of all topical reports currently accepted for review by the NRC.

This report will be published semiannually. Volume 8, No. 1, published in May 1994, was the only edition published for that year. Each sponsoring organization with one or more topical reports accepted for review receives copies.

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TOPICAL REPORT REVIEW STATUS

NUCLEAR REGULATORY COMMISSION LICENSING TOPICAL REPORT PROGRAM

Under the Nuclear Regulatory Commission (NRC) licensing topical report program, industry organizations may, on their own volition or at the request of the NRC staff, submit reports to the NRC on specific safety-related subjects and have them reviewed independently of any construction permit or operating license review. The purpose of this program is to minimize time and effort required of both industry and the NRC by providing for a single review and approval of the safety-related subject with subsequent referencing in licensing actions, rather than repetitive reviews of the same subject. The reports currently accepted for review are given in the appendix.

LICENSING TOPICAL REPORT PROGRAM MANAGEMENT

A topical report program manager (PM) manages and coordinates the overall program. Each review branch with responsibility for the detailed technical review of a specific report assigns review specialists who report to their own management on the detailed technical review matters.

PROCEDURE FOR SUBMITTING LICENSING TOPICAL REPORTS

Submittal of Reports for Review

A report (and requisite number of copies, as discussed below) submitted under the NRC licensing topical report program must be accompanied by a letter of transmittal. The first paragraph of this letter must include a statement that the applicant is submitting a licensing topical report under the NRC licensing topical report program for review and acceptance for referencing in licensing actions. This paragraph must specify the report identifier, title, and issue date. The transmittal letter and report should be addressed to the Document Control Desk, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; ATTN: Chief, Planning, Program and Management Support Branch. If a report is not sent to this address, there is no assurance that it will be entered into the agency's workload information and scheduling program database for tracking.

The transmittal letter should also include any pertinent information about the timeframe of the review and first referencing action. All correspondence with the NRC concerning licensing topical reports or special reports should include the report identifier specified by the submitting organization in the upper right-hand corner of the transmittal letter.

Copy Requirements

Licensing topical reports must be submitted in hard copy in the following quantities. Additional copies will be provided to the NRC staff upon request.

<u>Report Classification</u>	<u>Hard Copy</u>
Nonproprietary reports	23
Proprietary reports	
Proprietary version	23
Nonproprietary version	12

These quantity requirements pertain to all review submittals including initial submittals, revisions, supplements, responses to requests for additional information, and NRC-approved versions.

FORMAT OF LICENSING TOPICAL REPORTS

Report Identifier

Each licensing topical report should have a unique alphanumeric identifier for filing and reference purposes that is assigned to it by the submitting organization. The report identifier should be typed in the upper right-hand corner of the first page or cover page of all documents relating to the report.

Each organization submitting licensing topical reports should consult with the NRC topical report PM to ensure that the identifier it selects does not conflict with that of another organization and to receive an NRC technical assistance control (TAC) number. The identifier for all proprietary reports should have "-P" or "(P)" at the end. Each nonproprietary version of a proprietary report should have the same identifier as the corresponding proprietary report except that "-NP" or "(NP)" should be included at the end. NRC-approved versions of topical reports should have "-A" after the identifier ("-P-A" for approved proprietary reports).

The initial submittal and all responses to NRC comments or requests for additional information regarding a specific licensing topical report should be identified by both the NRC TAC number and the submitting organization's identifier for that report followed by "Response to Comment," "Response to Request for Additional Information by the NRC Staff," or if appropriate, "Response by Appendix or Supplement to the Original Report." The use of an addendum is discouraged. It is possible that only a revision to the original report can constitute a response. This is acceptable when appropriate. However, each revision will entail a new review, and the old review will be closed out.

Abstract

Each report should include an abstract no longer than one page that summarizes the contents of the report and the conclusions reached.

Introduction

Each report should have an introductory section that gives the purpose of the report and clearly defines its scope and applicability. Any limitations or restrictions on the use of the report or its results or conclusions as determined by either the staff or the sponsoring organization should also be listed in this section.

Body of Report

The body of each report may be organized at the discretion of the sponsoring organization to suit its needs and the subject matter of the report. It is recommended that long tabulations of data such as test results, computer program descriptions, detailed technical analyses, or derivations be included as appendices.

References

Each report should include a list of all pertinent references.

CRITERIA FOR ACCEPTING NEW LICENSING TOPICAL REPORTS FOR REVIEW

The cognizant technical review organization screens reports submitted for review under the licensing topical report program in accordance with the following criteria. No new report submitted by any organization will be accepted as a licensing topical report unless the cognizant technical review organization determines that it qualifies as such. In making such a determination, all four of the following criteria should be met:

- (1) The report deals with a specific "safety-related" subject regarding a nuclear power plant that requires a safety assessment by the NRC staff, such as component design, analytical models or techniques, or performance testing of components and/or systems that can be evaluated independently of any specific license application.
- (2) The report is, or is expected to be, referenced in a number of license or standardized reference design approval applications.
- (3) The report contains complete and detailed information on the specific subject presented. Conceptual or incomplete preliminary information will not be reviewed.
- (4) NRC approval of the report will result in increased efficiency of the review process for applications that reference the report.

Licensing topical reports that present a new design or procedure not currently addressed in any license or standardized reference design approval application require special consideration. In support of such a report, the sponsoring organization must submit documentation from at least two potential applicants of their intent to reference the report before the NRC commits resources to perform the review. It is not the intent of the licensing topical report program to provide engineering evaluations of feasibility, economics, or desirability of material submitted that could directly or indirectly be used in sales promotion efforts.

Exceptions to these acceptance criteria may be allowed. These exceptions must be based on an NRC decision that it is in the public interest to evaluate a proposed licensing topical report. The sponsoring organization should provide the NRC with justification for such exceptions. This may include such items as cost savings to the industry if implemented, or a consideration of whether

the evaluation would contribute to the formation of a broader base for resolving a present or developing safety-related subject generally evidenced by experience from operating nuclear power plants. As an example, a proposed licensing topical report on a new pump design might be accepted for review if experience at operating nuclear power plants indicated that currently installed pumps might be expected to infringe on the margins of safety to an unacceptable degree, and the new pump design might reasonably be expected to remedy the problem.

These exceptions will include a determination that NRC staff resources expended in the review of the licensing topical report will provide a foreseeable benefit equal to or greater than the impact from redirection and reduction of resources committed to other ongoing license actions, licensing topical report reviews, or generic technical issues.

When a sponsoring organization is planning a report that it believes can qualify as a licensing topical report, it should contact the topical report PM well in advance of the planned submittal to determine if the proposed report will satisfy the definition of a licensing topical report and, if so, to clarify the requirements for preparing and submitting the report. A meeting might be appropriate to discuss these issues. The topical report PM, in consultation with the staff technical specialists and management, will make a timely determination on the qualifications of the proposed report. If the report qualifies as a licensing topical report, the sponsoring organization may then formally submit it for review.

The NRC may also find that addressing a safety-related matter in a licensing topical report is desirable. In such a case, NRC management will contact the sponsoring organization and discuss the preparation of such a report. If the organization agrees to prepare it, they will formally submit it for review.

Other types of reports that do not meet the criteria for licensing topical reports may be submitted as special reports.

If accepted for review under the licensing topical report program, the report will be listed in the appendix to this NUREG series report. If the report is rejected for review, the sponsoring organization will be notified of this decision.

HANDLING OF PROPRIETARY REPORTS

When an organization submits a licensing topical report for NRC review pursuant to this NUREG series report and believes that portions of the report contain confidential business (proprietary) information, it must submit at least 12 copies of a nonproprietary version of the proprietary report at the same time it submits the proprietary report. The proprietary report must show which portions are proprietary by the use of italics or margin lines or by underscoring or bracketing the proprietary material. A nonproprietary report should show what information has been deleted from the proprietary version because the submitting organization considers it to be proprietary. For example, if brackets are used in the proprietary version to show what portion of the information is considered proprietary, the nonproprietary version

should have a blank portion between the brackets to show that information was removed. Upon submittal of a properly marked licensing topical report (see Section 2.790 of Title 10 of the Code of Federal Regulations) considered and justified as proprietary, the NRC will determine whether or not the information should be withheld from public disclosure and will notify the submitting organization accordingly.

CORRESPONDENCE FROM SPONSORING ORGANIZATION PERTAINING TO REPORTS

All correspondence regarding licensing topical reports should be addressed to the Document Control Desk, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; ATTN: Chief, Planning, Program and Management Support Branch, and should contain both the appropriate report identifier and NRC TAC number in the upper right-hand corner.

NRC REQUESTS FOR ADDITIONAL INFORMATION DURING REVIEW CYCLE

The NRC may request additional information on a licensing topical report under review either during a meeting or by letter. The chief of the technical review branch will transmit to the sponsoring organization written requests for additional information on licensing topical reports. The sponsoring organization will submit its responses to these requests and to staff comments during the review as information supplementary to the original report. The technical review branch will schedule meetings with the sponsoring organization to discuss the contents of a licensing topical report.

TRANSMITTAL OF NRC EVALUATION OF REPORTS

When the NRC determines that a licensing topical report is acceptable for referencing, it will give the extent of and the conditions for acceptance, if any, in a letter transmitting the results of the evaluation. For a proprietary report, the transmittal letter will state that both proprietary and nonproprietary versions must be referenced in future license applications. The NRC evaluation and letter of transmittal will be incorporated in the accepted version of the report when submitted.

PUBLICATION OF APPROVED VERSION OF REPORTS

After the NRC accepts a licensing topical report for referencing, the sponsoring organization should publish an approved version. A copy of NRC's transmittal letter and its evaluation report should be inserted immediately after the title page of the approved version.

PLACEMENT OF REPORTS AND ASSOCIATED DOCUMENTS IN PUBLIC DOCUMENT ROOM

All nonproprietary correspondence regarding the review of licensing topical reports, all nonproprietary licensing topical reports, and all nonproprietary versions of proprietary reports will be placed in the NRC Public Document Room (PDR) located at 2120 L Street, N.W., Washington DC; telephone (202) 634-3273. The sponsoring organization and the NRC should exercise care to ensure that any information identified as proprietary by the sponsoring organization is not used in any of the above correspondence and in licensing topical report

evaluations. Licensing topical reports are not placed in local PDRs; however, nonproprietary versions are available on request.

REVISIONS TO REPORTS

After a licensing topical report has been found acceptable for referencing, the sponsoring organization occasionally might need to add supplementary information to keep it up to date. This is different from the situation where the report has become obsolete and must be withdrawn. A proposed revision to an approved report is submitted to and reviewed by the NRC in accordance with the same requirements and procedures as those that apply to a new licensing topical report. The revision will have the same identifier as the base report with the suffix "Revision 1 or 2," etc. However, the NRC topical report PM will issue a new NRC TAC number. When a revision to an accepted report is submitted for review, the "-A" designation will no longer be valid until the revision itself has been reviewed and accepted.

When the NRC determines that the revised or supplementary material is acceptable for referencing, the sponsoring organization will submit the required number of complete new reports incorporating the revised or supplementary information. The revised report must contain a note on the cover page stating that it supersedes and replaces all earlier versions of the same numbered report. The superseded reports will not be returned to the sponsoring organization.

WITHDRAWAL OF REPORTS

After a certain amount of time has elapsed, a licensing topical report may become obsolete, as determined by either the NRC or the sponsoring organization, because it does not adequately address the current state of the technology, or it is no longer applicable because NRC criteria or regulations have changed, or for other reasons. In this event, the sponsoring organization may, on its own volition, request that the report be withdrawn or the NRC may advise the sponsoring organization to do so. On receipt of a letter from a sponsoring organization requesting withdrawal of a licensing topical report, the NRC will categorize it as "withdrawn." Withdrawn reports are not acceptable as references in license applications. The report may be replaced by an updated or new licensing topical report that has a new identifier and new NRC TAC number to avoid confusion with the withdrawn report.

APPENDIX

TOPICAL REPORTS ACCEPTED FOR REVIEW BY THE NRC STAFF

EXPLANATION OF APPENDIX

TASK ID	Technical Assignment Control (TAC) number used for tracking purposes
REV	Name of the organization reviewing the topical report
TITLE	Name of the topical report
APPL DATE	Date of the letter transmitting the topical report to NRC
SCHEDULE ACT START	Planned or actual technical review start date
TR COMP	Planned or actual technical review completion date
ACTION COMPLETE	Date of document which closes action on the topical report (NRC letter of approval or rejection, sponsoring organization letter of withdrawal)
ACT HOURS	Staff hours that have been charged in the review of a topical report
SCH REM HOURS	Remaining number of <u>scheduled</u> staff hours to complete review of topical report
APPL ACCESSION	Accession number given the letter transmitting the topical report when submitted to NRC for review
X =	Cancelled
H =	Hold
L =	Target Date Passed
C =	Complete
X(CR) =	Scheduled at a constant rate of x hours per week; i.e., 4(CR) indicates 4 hours per week
Unassigned =	Not yet assigned to a reviewer in organization shown
Suppressed =	Reviewer currently not on NRR staff

NOTES:

- The appendix does not reflect contractor costs.
- The estimates shown in the appendix are based on workload and resource projections that are focused on the next 6 months. Priority of a review task is determined primarily on the basis of safety significance, risk considerations, and operational impact. Schedules are subject to change in the event of higher priority, safety-related issues.

Report Date 01/31/95

Topical Manager Report (Condensed)

For ABB COMBUSTION ENGINEERING

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch.	
				Act Start	TR Complete				Rem. Hours	Appl. Accession
M80895		ABB REPORT - RPA 89-053 "ABB HIGH WORTH RODS, ROD DROP ACCIDENT ANALYSIS"	(02/09/90)	(--/--/--)	(--/--/--)	(--/--/--)				9002060263
M81374		CEN-403 - ESFAS SUBGROUP RELAY TEST INTERVAL EXTENSION	(07/31/91)	(09/07/91)	(--/--/--)	(--/--/--)	318	92		
	HICBA			01/08/94	03/31/95		224	66		
	HICBA			11/09/91	02/15/95		39	26		
	HICBA			09/07/91	07/16/93X		55	---		
M82272		CEN-405-P, REV.1-P - APPLI OF REACTOR VESS SURVEILL DATA FOR EMBRITT MGMT	(12/06/91)	(12/28/91)	(09/14/93C)	(--/--/--)	109	0	9112090388	
	EMCBA			12/28/91	09/14/93C		109	---		
M85911		CENPD-282 (P), VOL. 4 - TECHNICAL MANUAL FOR THE CENTS CODE	(02/17/93)	(06/11/94)	(--/--/--)	(--/--/--)	50	8	9303030182	
	SRXBC			06/11/94	02/17/95		50	8		
M86126		CENPD-283-P/NP - BWR EMERGENCY CORE COOLING EVALUATION MODEL CODE SENSITIVITY FOR SVEA-96 FUEL.	(03/30/93)	(04/24/93)	(--/--/--)	(--/--/--)	53	43	9304020195	
	SRXBA			04/24/93	09/30/95		53	43		

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NUREG-0390

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Topical Manager Report (Condensed)

For ABB COMBUSTION ENGINEERING

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M04685		CENPD-212-NP - INSTRUMENT QUALIFICATION ENVIRONMENTAL QUALIFICATION OF C-E INSTRUMENTATION EQUIPMENT, PARTS ONE.	(07/01/77)	(--/--/--)	(--/--/--)	(--/--/--)				
M47020		CENPD-172A-NP - PLANT PROTECTION SYSTEM WITH INTEGRATED AUTOMATIC TEST SYSTEM (SSPPS) INTERCONNECT DES	(12/15/75)	(--/--/--)	(--/--/--)	(--/--/--)				
M72033		CEN-381-P - LOW TEMP OVERPRESSURIZATION TRANSIENT P-T LIMIT FOR DETERMINATION LOW TEMP OP PROTE	(01/12/89)	(10/27/90)	(06/30/91C)	(--/--/--)	152	0	8901250404	
	EMCBA			05/04/91	06/30/91C		152	---		
M75965		CEN RPA 89-112 - ABB ATOM CONTROL ROD DROP ACCIDENT ANALYSIS METHODOLOGY FOR BWRS	(01/31/90)	(07/13/91)	(--/--/--)	(--/--/--)	90	13	9002060263	
	SRXBC			07/13/91	05/01/95		90	13		
M75966		CEN-00-000 RPA-89-053 P - ABB ATOM HIGH WORTH CONTROL RODS FOR BWRS ROD DROP ACCIDENT ANALYSIS	(01/31/90)	(08/24/91)	(--/--/--)	(--/--/--)	52	51	9002060263	
	SRXBC			08/24/91	06/30/95		52	51		

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Topical Manager Report (Condensed)

For ABB COMBUSTION ENGINEERING

TaskID	Rev.	Title	Appl Date	Schedule		Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete				
M86193		CEN-420-P, VOL 1, PART 1 - SMALL BREAK LOCA REALISTIC EVALUATION MODEL.	(03/05/93)	(07/24/93)	(--/--/--)	(--/--/--)	11	0	9304120127 9310080206
	SRXBB			07/24/93	09/29/93X		11	---	
M87733		CEN-405-P REV 2 - APPLICATION OF REACTOR VESSEL SURVEILLANCE DATA FOR EMBRITTELEMENT MANAGEMENT	(08/06/93)	(--/--/--)	(--/--/--)	(--/--/--)			9308180251
M87910		CENPD-289-P, JUSTIFYING USE OF INSERT REPLACEMENT RODS IN ABB CENF FUEL ASSEMBLIES.	(09/30/93)	(01/01/94)	(--/--/--)	(--/--/--)	70	29	9310050224
	SRXBC			01/01/94	12/31/95		70	29	
M88025		CENPD-284-P - CONTROL ROD DROP ACCIDENT ANALYSIS METHODOLOGY FOR BWR'S: SUMMARY AND QUALIFICATION	(10/01/93)	(03/26/94)	(--/--/--)	(--/--/--)	46	96	9310140114
	SRXBC			03/26/94	06/02/95		46	96	
M88029		CEN-420-P, VOL 1 - SMALL BREAK LOCA REALISTIC EVALUATION	(09/29/93)	(11/06/93)	(--/--/--)	(--/--/--)	16	60	9310080206
	SRXBB			11/06/93	03/28/97		16	60	

Report Date 01/31/95

Topical Manager Report (Condensed)

For ABB COMBUSTION ENGINEERING

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M88447		CEN-420-P, VOL II, SUPPLS. 1 & 2 AND VOL 3 - SMALL BREAK LOCA REALISTIC EVALUATION MODEL	(12/03/93)	(--/--/--)	(--/--/--)	(--/--/--)	0	106	9312210111	
	SRXBB			--/--/--	01/31/97		0	106		
M89487		CENPD-290-P - ABB BWR GENERIC CONTROL ROD DESIGN METHODOLOGY	(02/16/94)	(08/06/94)	(--/--/--)	(--/--/--)	28	45	9402280297	
	SRXBC			08/06/94	12/30/95		28	45		
M89663		CENPD-288 - ABB SEISMIC/LOCA EVALUATION METHODOLOGY FOR BWR FUEL	(05/26/94)	(09/03/94)	(--/--/--)	(--/--/--)	18	105	9406060335	
	SRXBC			09/03/94	07/14/95		18	105		
M90165		CENPD-292-P - BISON; ONE-D DYNAMIC ANALYSIS CODE FOR BWRS SUPP. 1	(08/02/94)	(10/01/94)	(--/--/--)	(--/--/--)	34	118	9408150107	
	SRXBA			10/01/94	06/30/95		34	118		
M90188		CENPD-285 - FUEL ROD DESIGN METHODS FOR BWRS	(06/08/94)	(09/03/94)	(--/--/--)	(--/--/--)	18	104	9406150153	
	SRXBC			09/03/94	06/02/95		18	104		

Report Date 01/31/95

Topical Manager Report (Condensed)

For ABB COMBUSTION ENGINEERING

TaskID	Rev.	Title	Appl Date	Schedule		Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete				
M90189		CENPD-287 - FUEL ASSEMBLY MECHANICAL DESIGN METHODOLOGY FOR BWRS	(06/17/94)	(09/03/94)	(--/--/--)	(--/--/--)	17	58	9406270348
	SRXBC			09/03/94	05/26/95		17	58	
M90850		CENPD-293-P - BOILING WATER REACTOR EMERGENCY COOLING SYS. EVAL. MODEL: SUPPL. 1 TO CODE DESCRIPTION & QUAL.	(08/26/94)	(11/26/94)	(--/--/--)	(--/--/--)	31	200	9412190234
	SRXBA			11/26/94	06/30/95		31	200	
M91197		CENPD-300-P - REFERENCE SAFETY REPORT FOR BWR RELOAD FUEL	(12/08/94)	(01/07/95)	(--/--/--)	(--/--/--)	12	93	9412160044
	SRXBA			01/07/95	10/30/95		12	93	

Report Date 01/31/95

Topical Manager Report (Condensed)

For BABCOCK & WILCOX

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M04645		BAW-10060-NP - REACTOR INTERNALS DESIGN/ANALYSIS FOR NORMAL UPSET & FAULTED CONDITIONS	(06/30/77)	(--/--/--)	(--/--/--)	(--/--/--)				
M75040		BAW-2086 - B&W J-R TEST PROCEDURE FOR WEDGE OPENING LOADED (WOL) FRACTURE TOUGHNESS SPECIMENS	(08/05/89)	(02/09/91)	(02/11/91C)	(--/--/--)	49	0	8908080117	
	EMCBA			02/09/91	02/11/91C		49	---		
M81851		BAW-10084-P, REV. 3 - PROGRAM TO DETERMINE IN-REACTOR PERFORMANCE OF B&W FUELS. JULY 1991	(09/09/91)	(10/31/92)	(--/--/--)	(--/--/--)	47	32	9110030218	
	SRXBC			10/31/92	06/30/95		47	32		
M84643		BAW-10164-P, REV.2 - ADVANCED COMPUTER PROGRAM FOR LIGHT WATER REACTOR LOCA AND NON-LOCA TRANSIENT ANALYSIS AUG.1992	(09/18/92)	(06/26/93)	(--/--/--)	(--/--/--)	88	9	9209250268	
	SRXBB			06/26/93	02/28/95		88	9		
M84800		BAW-10164-P, REV. 3 - RELAP/MOD 2	(10/08/92)	(02/20/93)	(--/--/--)	(--/--/--)	140	9	9210280244	
	SRXBB			06/26/93	02/28/95		129	9		
	SRXBB			02/20/93	10/31/93X		11	---		

Report Date 01/31/95

Topical Manager Report (Condensed)

For BABCOCK & WILCOX

TaskID	Rev.	Title	Appl Date	Schedule		Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete				
M84985		BAW-10168-P, REV.2 - RSG LOCA, BWNT LOCA EVALUATION MODEL FOR RECIRCULATING STEAM GENERATOR PLANTS	(11/10/92)	(02/27/93)	(--/--/--)	(--/--/--)	54	60	9211200193
	SRXBB			10/02/93	06/30/95		45	60	
	SRXBB			02/27/93	08/01/94X		9	---	
M85120		BAW-10186-P - EXTENDED BURNUP EVALUATION, NOVEMBER 1993	(11/24/92)	(05/22/93)	(--/--/--)	(--/--/--)	108	0	9212030353
	SRXBC			05/22/93	12/31/95		108	0	
M87102		BAW-10191-P - STAR SYSTEM COMPONENTS FOR REACTOR PROTECTION SYSTEM DIGITAL UPGRADES.	(07/19/93)	(08/14/93)	(--/--/--)	(--/--/--)	306	193	9307260172
	HICBA			10/02/93	06/30/95		62	44	
	HICBA			08/14/93	06/30/95		168	115	
	HICBA			08/14/93	06/30/95		26	35	
	HICBA			04/30/94	12/13/95X		50	---	
M87805		BAW-10189-P - CHF TESTING AND ANALYSIS OF THE MARK-BW FUEL ASSEMBLE DESIGN, 9/93.	(09/20/93)	(10/09/93)	(01/03/95C)	(--/--/--)	134	0	9309240121
	SRXBC			10/09/93	01/03/95C		134	---	

Report Date 01/31/95

Topical Manager Report (Condensed)

For BABCOCK & WILCOX

TaskID	Rev.	Title	Appl Date	Schedule		Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete				
M88739		BAW-2181-NP, CRAFT 2 - RELAPS/MOD 2; COMPARISON FOR SMALL BREAK LOCA ACCIDENT ANALYSIS	(02/19/93)	(02/12/94)	(--/--/--)	(--/--/--)	7	62	9303090099
	SRXBB			02/12/94	03/31/96		7	62	
M89400		B&W 10192P - LOCA EM FOR OTSG PLANTS.	(02/15/94)	(--/--/--)	(--/--/--)	(--/--/--)	0	60	9404290115
	SRXBB			--/--/--	03/31/96		0	60	

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For BWR OWNER'S GROUP

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M83823		BWROG - BWR BELTLINE MATERIAL UPPER SHELF ENERGY ESTIMATION METHODS (GE-NE-523-18-1191)	(06/12/92)	(06/27/92)	(06/24/93C)	(--/--/--)	133	0		
	EMCBA			06/27/92	08/11/92C		11	---		
	EMCBA			07/11/92	12/31/92C		88	---		
	EMCBA			07/04/92	06/24/93C		14	---		
	PD11			07/18/92	05/11/93X		20	---		
M91297		BWROG ABB - CE LTSSS OPTION III SYSTEM DEVELOPMENT	(--/--/--)	(--/--/--)	(--/--/--)	(--/--/--)	0	0		
	HICBA			--/--/--	--/--/--		0	0		

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For DUKE POWER

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M85181		DPC-NF-2005-P - THERMAL/HYDRAULIC STATISTICAL CORE DESIGN METHODOLOGY	(09/28/92)	(01/16/93)	(--/--/--)	(--/--/--)		142	2	9210090053
	DSSA			01/16/93	05/03/93X			9	---	
	SRXBB			07/03/93	01/30/95L			133	2	
M88082		DUKE-2007 P - FUEL RECONSTITUTION ANALYSIS METHODOLOGY	(09/23/93)	(12/25/93)	(--/--/--)	(--/--/--)		49	50	9309300259
	PD23			12/25/93	--/--/--H			6	0	
	SRXBC			01/01/94	06/30/95			43	50	

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For ELECTRIC POWER RESEARCH INSTITUTE

TaskID	Rev.	Title	Appl Date	Schedule		Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete				
M80205		EPRI-NP-6628 - PROCEDURE FOR SEISMIC EVALUATION AND DESIGN OF SMALL BORE PIPING (NCIG-14)	(--/--/--)	(05/04/91)	(--/--/--)	(--/--/--)	367	0	
	-----			11/09/91	12/13/91C	Suppressed	94	---	
	-----			05/18/91	--/--/--X	Suppressed	30	---	
	EMEBA			05/18/91	--/--/--	Unassigned	0	0	
	EMEBA			05/04/91	04/18/94C		243	---	
M88026		EPRI-TR-102470-P - HIGH FREQUENCY SEISMIC EFFECTS	(06/01/93)	(10/30/93)	(--/--/--)	(--/--/--)	259	117	
	-----			10/30/93	06/30/94C	Suppressed	61	---	
	ECGB			04/09/94	10/20/95		6	39	
	ECGBA			11/06/93	10/06/94C		41	---	
	ECGBA			--/--/--	10/07/94C		0	---	
	ECGBB			04/09/94	08/30/95		151	78	
M88898		EPRI MOV - TOPICAL REPORT REVIEW	(02/22/94)	(06/11/94)	(--/--/--)	(06/01/95T)	79	110	
	EMEBB			06/18/94	06/30/95		63	110	
	PD33			07/09/94	--/--/--H		9	0	
	PD33			06/11/94	--/--/--H		7	0	
M91235		EPRI-102323 - Guidelines for Electromagnetic Interference for Testing Power Plants	(12/19/94)	(01/14/95)	(--/--/--)	(--/--/--)	35	290	9412270123
	HICBC			01/14/95	08/11/95		35	290	

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For GENERAL ELECTRIC

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M64221		NEDC-31336-P GE INSTRUMENT SET POINT METHODOLOGY	(11/19/86)	(03/16/91)	(08/26/92C)	(--/--/--)		251	0	8612040106
				03/07/92	01/30/93X	Suppressed		2	---	
				03/16/91	02/18/92X	Suppressed		151	---	
	DSSA			02/29/92	08/26/92C			63	---	
	HICBA			08/01/92	08/26/92C			35	---	
M77253		NEDE-24011-P-A, AM 23 - GE STANDARD APPLICATION FOR REACTOR FUEL (GESTAR-11)	(01/19/90)	(01/04/92)	(--/--/--)	(--/--/--)		11	0	
	SRXB			--/--/--	10/20/90C			0	---	
	SRXBC			01/04/92	11/25/96			11	0	
M82663		NEDC 31984P - GENERIC EVAL. OF GE BWR POWER UPRATE, SUPPL 1, DTD 10/91	(10/22/91)	(02/01/92)	(07/31/92C)	(--/--/--)		132	0	9110290418
				02/01/92	01/01/94X	Suppressed		89	---	
	EMEBA			11/14/92	07/31/92C			15	---	
	SRXBA			02/08/92	04/03/92C			28	---	

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For GENERAL ELECTRIC

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M83089		NEDC-32013P - SYSTEM ANALYSES FOR ELIMINATION OF SELECTED RESPONSE TIME TESTING REQUIREMENTS	(03/27/92)	(04/11/92)	(--/--/--)	(--/--/--)		804	3	9204170140
	HICBA			04/11/92	12/13/95X			506	---	
	HICBA			11/19/94	12/13/95X			17	---	
	HICBA			04/16/94	12/13/95X			10	---	
	SRXBA			10/02/93	01/31/95			269	3	
	SRXBB			12/18/93	01/07/94X			2	---	
M84984		NEDC-31984-P,SUPP 2 - GE BWR POWER UPRATE, OCTOBER 1992	(11/06/92)	(12/26/92)	(--/--/--)	(--/--/--)		55	0	9211120151
	-----			12/26/92	01/01/94X	Suppressed		55	---	
M85776		NEDC-32160-P - BWROG "CALIBRATION INTERVAL EXTENSION"	(01/08/93)	(05/01/93)	(--/--/--)	(--/--/--)		182	184	9302040080
	HICBA			11/05/94	06/30/95			6	35	
	HICBA			05/01/93	06/30/95			176	150	
M85913		NEDE-32176-(P) - TRACG, MODEL DESCRIPTION, 2/93.	(02/23/93)	(03/13/93)	(--/--/--)	(--/--/--)		380	302	9303030220
	SASG			03/20/93	01/31/97			62	253	
	SCSB			09/18/93	07/05/95			7	0	
	SRXBA			03/13/93	12/31/95			311	49	

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TaskID	Rev.	Title	Appl Date	Schedule		Action Complete	Act. Hours	Sch.	
				Act Start	TR Complete			Rem. Hours	Appl. Accession
M85915		NEDE-32178-(P) - APPLIC. OF TRACG MODEL TO SBWR LICENSING SAFETY ANALYSES, 2/93.	(02/25/93)	(03/20/93)	(--/--/--)	(--/--/--)	350	866	9303030233
	SASG			03/20/93	08/22/97		46	283	
	SCSB			09/18/93	07/05/95		7	0	
	SRXBA			03/20/93	06/06/96		298	583	
M87103		NEDE-32177-P - TRACG QUALIFICATION, REV 1, JUNE 1993.	(07/12/93)	(08/28/93)	(--/--/--)	(--/--/--)	258	275	9307270022
	SCSB			11/19/94	09/30/95		12	76	
	SRXBA			08/28/93	12/31/95		244	199	
	TERBB			09/24/94	--/--/--		2	0	
M87911		NEDC-31858-P, REV 2 - BWROG RPT FOR INCREASING MSIV LEAKAGE RATE LIMITS & ELIMINATION OF LEAKAGE CONTROL SYSTEMS	(09/01/94)	(10/23/93)	(--/--/--)	(--/--/--)	103	25	
	ECGBA			--/--/--	10/04/94C		0	---	
	EMEBA			10/30/93	02/22/94C		63	---	
	PD11			10/23/93	05/27/94C		3	---	
	PD13			12/04/93	06/20/94X		1	---	
	SCSB			12/17/94	--/--/--		5	0	
	SPLBA			11/06/93	04/07/95		32	25	

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				Act Start	TR Complete				Rem. Hours	Appl. Accession
-----	-----	-----	----	-----	-----	-----	-----	-----	-----	-----
M89222		NEDC-32339P & NEDO-32339 - REACTOR STABILITY LONG TERM SOLUTION: ENHANCED OPTION 1-A	(04/14/94)	(04/30/94)	(--/--/--)	(--/--/--)		123	144	
	HICBA			01/14/95	12/31/95			1	132	
	SRXBC			07/16/94	01/31/95			16	3	
	SRXBC			05/21/94	01/31/95			50	8	
	SRXBC			04/30/94	03/15/95			38	0	
	SRXBC			05/14/94	01/31/95			18	1	
M89259		NEDC-32264, Rev 1 - APPLICATION OF PSA TO GL-89-10 IMPLEMENTATION (BWROG)	(02/03/94)	(04/23/94)	(--/--/--)	(--/--/--)		152	14	
	EMEBB			04/23/94	03/31/95			30	14	
	PD33			01/07/95	--/--/--H			9	0	
	SPSBA			04/23/94	--/--/--H			113	0	
M89629		NEDO-31331 - PROCEDURE GUIDELINES MODS ADDRESSING ATWS STABILITY ISSUE	(04/18/94)	(06/18/94)	(--/--/--)	(--/--/--)		706	89	
	SPLBA			08/06/94	08/05/94C			8	---	
	SPSBA			08/06/94	10/12/94X			30	---	
	SPSBB			08/06/94	10/12/94X			15	---	
	SRXBC			06/18/94	03/15/95			437	33	
	SRXBC			06/18/94	03/15/95			13	8	
	SRXBC			06/18/94	03/15/95			164	20	
	SRXBC			06/18/94	03/15/95			35	18	
	TERBB			08/06/94	03/31/95			4	10	

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NUREG-0390

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TaskID	Rev.	Title	Appl Date	Schedule		Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete				
M90169		NEDC-32343P - CRITERIA FOR DELETING NON-SAFETY BASIS NRC COMMITMENTS AND USAR INFORMATION	(07/27/94)	(10/01/94)	(10/06/94C)	(--/--/--)	29	0	9407290197
	OTSB			10/01/94	10/06/94C		29	---	
M90468		NEDC-32339-P - REACTOR STABILITY LONGTERM SOLUTION: ENHANCEDOPTION I-A	(04/14/94)	(10/29/94)	(--/--/--)	(--/--/--)	14	4	
	SRXBC			10/29/94	02/28/95		14	4	
M90616		NEDC-32410-P - NUCLEAR MEASUREMENT ANALYSIS AND CONTROL PRNM	(09/29/94)	(11/19/94)	(--/--/--)	(--/--/--)	72	191	9410070149
	HICBA			11/19/94	12/31/95		65	191	
	PD23			11/26/94	--/--/--		7	0	
	TSIBB			--/--/--	--/--/--		0	0	

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Topical Manager Report (Condensed)

For ROBERT L. CLOUD & ASSOC., INC.

TaskID	Rev.	Title	Appl Date	Schedule			Sch.		
				Act Start	TR Complete	Action Complete	Act. Hours	Rem. Hours	Appl. Accession
M90271		RLCA/P94/04-94/009(P) - ROBERT L. CLOUD & ASSOC; SEISMIC STOPS - GAPPIPE METHODOLOGY	(07/22/94)	(10/22/94)	(--/--/--)	(--/--/--)	78	0	9409210303
	EMEBA			10/22/94	11/30/94L		78	0	

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Topical Manager Report (Condensed)

For SIEMENS

TaskID	Rev.	Title	Appl Date	Schedule		Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete				
M80400		ANF-90-145(P) VOLS 1 & 2 - RODEX3 FUEL ROD THERMAL-MECHANICAL RESPONSE EVALUATION MODEL, ADVANCED NUCLEAR FUEL CORP	(05/06/91)	(08/07/93)	(--/--/--)	(--/--/--)	92	4	9104230075
	SRXBC			08/07/93	01/31/95		92	4	
M81069		ANF-90-082(P) REV 1 - APPLICATION OF ANF DESIGN METHODOLOGY FOR FUEL ASSEMBLY RECONSTITUTION ADVANCE NUCLEAR FC, MAY	(06/06/91)	(02/29/92)	(--/--/--)	(--/--/--)	180	13	9106190217
	DSSA			02/29/92	--/--/--X		6	---	
	SRXBC			08/08/92	02/28/95		69	13	
	SRXBC			04/11/92	--/--/--X		105	---	
M81070		ANF-89-90(P) REV 1 - GENERIC MECHANICAL DESIGN CRITERIA FOR BWR FUEL DESIGNS, ADVANCED NUCLEAR FUEL CORP. APRIL 1991	(05/16/91)	(03/21/92)	(--/--/--)	(--/--/--)	278	2	9106030332
	SRXBC			03/21/92	01/30/95L		278	2	
M84245		EMF-92-116 P - GENERIC MECHANICAL DESIGN CRITERIA FOR PWR FUEL DESIGNS.	(08/03/92)	(05/07/94)	(--/--/--)	(--/--/--)	71	12	9208050140
	SRXBC			05/07/94	02/20/95		71	12	

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For SIEMENS

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M84718		EMF-92-139(P), VOL 3 - REALISTIC LOCA ECCS EVALUATION MODEL ASSESSMENT FOR PWR LARGE BREAK LOCA ANALYSIS	(09/14/92)	(03/26/94)	(--/--/--)	(--/--/--)		1	134	9210140232
	SRXBB			03/26/94	08/29/97			1	134	
M84811		EMF-92-139(P), VOL 2, - S-RELAP 5 REALISTIC LOCA ECCS EVALUATION MODEL FOR PWR LARGE BREAK LOCA ANALYSIS.	(10/09/92)	(11/01/92)	(--/--/--)	(--/--/--)		2	137	9210280154
	SRXB			11/01/92	11/16/92C			0	---	
	SRXBB			03/26/94	09/26/97			2	137	
M85182		EMF-92-139(P), VOL 1 - REALISTIC LOCA ECCS EVALUATION MODEL METHODOLOGY FOR PWR LARGE BREAK LOCA ANALYSIS, 12/92	(12/01/92)	(09/11/93)	(--/--/--)	(--/--/--)		36	140	9212040297
	SRXBB			09/11/93	09/26/97			36	140	
M87010		EMF-92-139(P), VOL 3, SUPP 2 - REAL LOCA ECCS EVAL MODEL ASSMT FOR PWR LARGE BREAK LOCA ANALYSIS ASSMT FOR CCTF TEST	(07/02/93)	(--/--/--)	(--/--/--)	(--/--/--)		0	137	9307070200
	SRXBB			--/--/--	09/15/97			0	137	

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M87011		EMF-92-139(P), VOL 3, SUP 1 - REAL LOCA ECCS EVAL MODEL ASSESS. FOR PWR LARGE BREAK LOCA ANALYSIS FLECHT SEASET 31	(07/02/93)	(--/--/--)	(--/--/--)	(--/--/--)	0	137	9307070200	
	SRXBB			--/--/--	09/15/97		0	137		
M87012		EMF-92-139-P, VOL 4, REALISTIC LOCA ECCS EVAL MODEL FOR PWR LARGE BREAK LOCA ANALYSIS-UNCERTAINTY ANALYSES, 7/93	(07/02/93)	(07/14/95)	(--/--/--)	(--/--/--)	0	270	9307070200	
	SRXBB			07/14/95	09/15/97		0	270		
M87013		ENF-92-139-P, V3, S7 - REALISTIC LOCA ECCS EVAL MODEL ASMT FOR PWR LARGE BREAK LOCA ANALYSIS, W 4 LOOP PWR SAM PROBL	(07/02/93)	(--/--/--)	(--/--/--)	(--/--/--)	0	130	9307070200	
	SRXBB			--/--/--	09/30/97		0	130		
M87014		EMF-92-139-P, V3,S6 - REALISTIC LOCA ECCS EVAL MODEL ASMT, PWR LARGE BREAK LOCA ANALYSIS-W 3-LOOP PWR SAM PROBLEM, 5	(07/02/93)	(--/--/--)	(--/--/--)	(--/--/--)	0	130	9307070200	
	SRXBB			--/--/--	09/30/97		0	130		

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For SIEMENS

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M87015		EMF-92-139-P, V3, S5 - REALISTIC LOCA ECCS EVAL MODEL ASMT FOR PWR LRGE BREAK LOCA ANALYSIS ASMT UPTF TEST 6&7, 6/93	(07/02/93)	(--/--/--)	(--/--/--)	(--/--/--)		0	120	9307070200
	SRXBB			--/--/--	09/30/97			0	120	
M87016		EMF-92-139-P V3,S4 - REALISTIC LOCA ECCS EVAL MODEL ASMT FOR PWR LARGE BREAK LOCA ANALYSIS LOFT TEST L2-6, 6/93	(07/02/93)	(--/--/--)	(--/--/--)	(--/--/--)		0	120	9307070200
	SRXBB			--/--/--	09/30/97			0	120	
M87017		EMF-92-139P V-3,S-3 - REALISTIC LOCA ECCS EVAL MODEL ASMT FOR PWR LARGE BREAK LOCA ANALYSIS-ASMT LOFT L2-5, 6/93	(07/02/93)	(--/--/--)	(--/--/--)	(--/--/--)		0	120	9307070200
	SRXBB			--/--/--	09/30/97			0	120	
M87907		EMF-93-164(P) - POWER DISTRIBUTION MEASUREMENT UNCERTAINTY FOR INPAX-W IN WESTINGHOUSE PLANTS, SEPT. 1993.	(10/01/93)	(10/23/93)	(11/17/94C)	(--/--/--)		32	0	9310140161
	SRXBC			10/23/93	11/17/94C			32	---	
M89662		EMF-93-177(P) - MECHANICAL DESIGN OF BWR CHANNELS, MAY 1994	(05/18/94)	(08/27/94)	(--/--/--)	(--/--/--)		12	46	9405250243
	SRXBC			08/27/94	06/30/95			12	46	

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For STONE & WEBSTER

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				Act Start	TR Complete					
M04723		SWECO-7703-NP - MISSILE-BARRIER INTERACTIONS	(09/01/77)	(--/--/--)	(--/--/--)	(--/--/--)				

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Topical Manager Report (Condensed)

For WESTINGHOUSE

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M00789		WCAP-08163-NP - REACTOR COOLANT PUMP INTEGRITY IN A LOCA	(09/01/73)	(--/--/--)	(--/--/--)	(--/--/--)				
M04246		WCAP-08510-NP - METHOD FOR FRACTURE MECHANICS ANALYSIS OF NUCLEAR REACTORS UNDER SEVERE THERMAL THRANSIENTS	(07/01/76)	(--/--/--)	(--/--/--)	(--/--/--)				
M04849		WCAP-09172-P & WCAP-9264 - AN N-16 TRANSIT TIME FLOW MEASUREMENT (TTFM) DESCRIPTION AND PERFORMANCE	(02/01/78)	(01/01/80)	(02/21/86C)	(--/--/--)	0	0		
	SRXBB			--/--/--	02/21/86C		0	---		
	SRXBC			01/01/80	02/06/81C		0	---		
M04882		WCAP-09292-NP - DYNAMIC FRACTURE TOUGHNESS OF ASME SA508 CLASS 2A ASME SA533 GRADE A CLASS 2 BASE & HEAT AFFECTED	(03/01/78)	(--/--/--)	(--/--/--)	(--/--/--)				
M61013		WCAP-10972-P - ABB ATOM-WESTINGHOUSE CORRELATION FOR PREDICTING CPR MARGINS IN BWRs	(01/29/86)	(03/23/91)	(--/--/--)	(--/--/--)	183	5	8602050335	
	SRXBC			03/23/91	02/15/95		183	5		
M64956		WCAP-10858-P-A(ADD 1) - AMSAC GENERIC DESIGN PACKAGE	(02/26/87)	(--/--/--)	(--/--/--)	(--/--/--)			8703100266	

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For WESTINGHOUSE

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				Act Start	TR Complete			Act. Hours	Rem. Hours	Appl. Accession
M66167		WCAP-11500-P - WESTINGHOUSE REFERENCE REPORT FOR BWR FUEL	(08/28/87)	(12/15/90)	(07/01/91C)	(--/--/--)				8709030549
M81078		WCAP-12909-P - WESTINGHOUSE ECCS EVALUATION MODEL: REVISED LARGE BREAK LOCA POWER DISTRIBUTION METHODOLOGY	(05/22/91)	(05/02/92)	(--/--/--)	(--/--/--)	78	3		9106100147
	SRXBB			05/02/92	02/10/95		78	3		
M81712		WCAP-12945-P, VOL 1 - WESTINGHOUSE CODE QUALIFICATION DOCUMENT FOR BEST ESTIMATE LOCA ANALYSIS	(08/31/91)	(10/19/91)	(--/--/--)	(--/--/--)	321	188		9108150130
	EELBA			--/--/--	09/30/93C		0	---		
	SRXBB			10/19/91	12/15/95		321	188		
M82099		WCAP-10924-P-REV. 2-VOLUME 2-ADDENDUM 3 - LARGE BREAK LOCA BEST ESTIMATE METHODOLOGY - UPPER PLENUM INJECTION MODEL	(04/30/91)	(12/07/91)	(--/--/--)	(--/--/--)	277	2		9104290088
	SRXBB			12/07/91	01/31/95		277	2		
M83235		WCAP-13360-P - WESTINGHOUSE DYNAMIC ROD WORTH MEASUREMENT TECHNIQUE, MAY 1992	(05/29/92)	(05/02/92)	(--/--/--)	(--/--/--)	67	120		9206050125
	SRXBA			05/02/92	06/30/95		67	120		

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				Act Start	TR Complete				
M83321		WCAP-12476 - EVALUATION OF LOCA DURING MODE 3 AND MODE 4 OPERATION FOR WESTINGHOUSE NSSS	(11/27/91)	(01/30/93)	(--/--/--)	(--/--/--)	26	36	
	SRXBB			04/24/93	09/29/95		16	36	
	TERBB			01/30/93	11/08/93C		10	---	
M83964		WCAP-12945-P VOL 1, REV. 1 - WESTINGHOUSE CODE QUALIFICATION DOCUMENT FOR BEST ESTIMATE LOCA	(06/10/92)	(12/17/94)	(--/--/--)	(--/--/--)	20	32	9206240140
	SRXBB			12/17/94	08/31/95		20	32	
M84324		WCAP-13246-P & WCAP-13412-NP - W-GOTHIC, A COMPUTER CODE FOR ANALYSES OF THERMAL HYDRAULIC TRANSIENTS FOR NUCLEAR PL	(07/31/92)	(--/--/--)	(--/--/--)	(--/--/--)			9208040270
M84796		WCAP-12945-P, VOL 2 - WESTINGHOUSE CODE QUALIFICATION DOCUMENT FOR BEST ESTIMATE LOCA ANALYSIS	(09/22/92)	(12/24/94)	(--/--/--)	(--/--/--)	2	32	9210220306
	SRXBB			12/24/94	08/31/95		2	32	
M85165		WCAP-12610-P - VANTAGE+ FUEL ASSEMBLY REFERENCE CORE REPORT	(06/13/90)	(05/01/93)	(--/--/--)	(--/--/--)	68	0	9007030117
	SRXBC			05/01/93	11/25/96		68	0	

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TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M85738		WCAP-13247-P - STEAM GENERATOR TUBE UNCOVERY ISSUE	(03/31/92)	(02/20/93)	(03/10/93C)	(--/--/--)	43	0	9312080088	
	SRXBB			02/20/93	03/10/93C		43	---		
M86251		WCAP-12945-P, VOL 3 - WESTINGHOUSE CODE QUALIFICATION DOCUMENT FOR BEST ESTIMATE LOCA ANALYSIS	(03/09/93)	(05/08/93)	(--/--/--)	(--/--/--)	40	32	9304130414	
	SPLBA			05/22/93	06/04/93X		9	---		
	SRXBB			05/08/93	08/31/95		31	32		
M86720		WCAP-13677-P - 10 CFR 50.46 EVALUATION MODEL RPT: WCOBRA/TRAC TWO-LOOP UPPER PLENUM IN MODEL UPDT TO SUPPORT ZIRLO/T	(05/25/93)	(06/19/93)	(11/26/93C)	(--/--/--)	47	0	9404220175	
	SRXBB			06/19/93	11/26/93C		47	---		
M86764		WCAP-13749-P - SAFETY EVALUATION SUPPORT THE CONDITION EXEMP OF THE MOST NEGATIVE EOL MOD TEMP COEFFICIENT MEASURE	(06/01/93)	(10/01/94)	(--/--/--)	(--/--/--)	42	50	9306080365	
	SRXBC			10/01/94	05/05/95		42	50		

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TaskID	Rev.	Title	Appl Date	Schedule		TR Complete	Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M87018		WCAP-12945-P, VOL 4&5 - WESTINGHOUSE CODE QUALIFICATION DOCUMENT FOR BEST ESTIMATE LOCA ANALYSIS	(06/23/93)	(07/31/93)	(--/--/--)	(--/--/--)		90	38	9307070227
	SRXBB			07/31/93	08/31/95			90	38	
M90784		WCAP-10054-P, ADD 2 & WCAP-10081-NP - WESTINGHOUSE NOTRUMP SBLOCA MODEL	(08/31/94)	(11/19/94)	(--/--/--)	(--/--/--)		16	34	9409080185
	SRXBB			11/19/94	10/31/95			16	34	
M91155		WCAP-13524-P, REV 1 - APPOLLO, A ONE-DIMENSIONAL NEUTRON DIFFUSION THEORY PROGRAM	(08/24/94)	(12/31/94)	(--/--/--)	(12/31/95)		20	44	9409140344
	SRXBC			12/31/94	12/29/95			20	44	

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For WESTINGHOUSE STEAM TURBINE-GENERATOR

TaskID	Rev.	Title	Appl Date	Schedule			Action Complete	Act. Hours	Sch. Rem. Hours	Appl. Accession
				Act Start	TR Complete					
M65737		WSTG-4-P/NP - ANALYSIS OF THE PROBABILITY OF THE GENERATION MISSILES FROM FULL INTEGRAL NUCLEAR LOW PRESSURE	(10/30/84)	(--/--/--)	(--/--/--)	(--/--/--)				

Total Number of Tasks = 103

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(See instructions on the reverse)

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10. SUPPLEMENTARY NOTES

11. ABSTRACT (200 words or less)

This report provides industry with procedures for submitting topical reports, guidance on how the U.S. Nuclear Regulatory Commission (NRC) processes and responds to topical report submittals, and an accounting, with review schedules, of all topical reports currently accepted for review by the NRC. This report is published semiannually.

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