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**EVALUATION OF THERMAL NEUTRON CROSS-SECTIONS  
AND RESONANCE INTEGRALS OF PROTACTINIUM,  
AMERICIUM, CURIUM, AND BERKELIUM ISOTOPES**

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Translated from *Jadernye Konstanty* 1993/1 p. 22-42

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ABSTRACT

Data on the thermal neutron fission and capture cross-sections as well as their corresponding resonance integrals are reviewed and analysed. The data are classified according to the form of neutron spectra under investigation. The weighted mean values of the cross-sections and resonance integrals for every type of neutron spectra were adopted as evaluated data.

**THERMAL CROSS-SECTIONS**

The experimental capture and fission cross-sections, referred to energy 0.0235 eV, may be divided into several groups depending on the shape of the spectrum of the neutrons under investigation:

- (1) Thermal cross-sections (let us denote them  $\sigma^0(n,\gamma)$  and  $\sigma^0(n,f)$ ), corresponding to neutron energy 0.0253 eV and measured by the time-of-flight method;
- (2) Effective thermal cross-sections obtained in neutron spectra with mean energy 0.0253 eV. Two types of such effective cross-sections may be distinguished:
  - (a) Thermalized cross-sections -  $\sigma_\gamma(\text{th})$  and  $\sigma_f(\text{th})$ . These thermal cross-sections, averaged over the Maxwellian spectrum having a mean neutron energy of 0.0253 eV, are measured principally in thermal columns by the activation method;

- (b) Effective cross-sections -  $\sigma_{\gamma}(\text{eff})$  and  $\sigma_f(\text{eff})$ . These are also thermal cross-sections, measured in reactor neutron spectra having a mean energy of 0.0253 eV. They are obtained either by the reactivity compensation method (pile oscillation) or by the method of isotope accumulation in the reactor.

The priority of the analysed cross-sections is determined by the methods employed for obtaining them, and these take the following order:

- (1) Time-of-flight method;
- (2) Activation method;
- (3) Reactivity compensation method;
- (4) Method of isotope accumulation.

The effective cross-section can be obtained from the relationship of the reactor characteristics:

$$\sigma(\text{exp}) = \sigma(\text{eff}) \sqrt{\frac{\pi T_0}{4T}} + RI \left[ \frac{\Phi_{\text{epi}}}{\Phi_{\text{th}}} : \ln \frac{E_{\text{th}}}{E_{\text{epi}}} \right],$$

where  $T_0$  is the reactor temperature,  $\Phi_{\text{epi}}/\Phi_{\text{th}}$  is the ratio of the epithermal and thermal fluxes  $E_{\text{th}}/E_{\text{epi}}$  is the ratio of the thermal and epithermal neutron energies,  $\sigma(\text{exp})$  is the cross-section measured in the given experiment.

It follows from the equation that  $\sigma(\text{eff})$  is very sensitive to the interface of thermal and epithermal energy, i.e. to the magnitude of  $E_{\text{cd}}$  - the cadmium shielding energy, particularly if the nucleus has levels in this region of neutron energies.

Ideally, all three types of thermal cross-sections should coincide. However, in practice considerable disparities are observed.

## RESONANCE INTEGRALS

The resonance integral (RI) is the conventional term used to describe the cross-section averaged over the epithermal neutron spectrum which has an energy distribution proportional to  $\sim 1/E$ .

$$RI = \int_{E_{cd}}^{\infty} \sigma(E) \frac{dE}{E} .$$

According to the method of obtaining it, RI can be classified as follows:

- (1)  $RI^0$  - calculated from differential data (resonance parameters);
- (2)  $RI(\text{eff})$  - obtained by the reactivity compensation method;
- (3)  $RI(\text{th})$  - measured by the activation method with a cadmium shield for more or less "pure" isotopes;
- (4)  $RI(\text{eff})$  - obtained from experiments on accumulation ("generation") of isotopes in reactor fuel.

Since the magnitude of RI depends on the integration limits, it is necessary to take detailed account of the effect of the lower,  $E_{cd}$ , and upper,  $E_k$ , boundaries of the neutron spectra.

RI is very sensitive to cadmium cut-off, particularly if the nucleus has resonances in this neutron energy region.

Table 1 presents some characteristics of the isotopes under consideration.

In the experiments,  $E_{cd}$  usually fluctuates from 0.3 to 0.68 eV.

It is evident that comparison of RI is valid only in the case of uniform values of  $E_{cd}$ .

The generally accepted value of  $E_{cd} = 0.5$  eV.

As a rule, in calculating  $RI_\gamma$  and  $RI_f$ , infinity is replaced by a finite value of  $E_k$  in the upper part of the spectrum, no correction being introduced for the contribution to  $RI$  of energies above  $E_k$ , and the quantity  $E_k$  itself is ignored. It is true that for capture and thresholdless fission processes the contribution to  $RI$  from neutrons with energy exceeding 20 MeV amounts to several barn, which are referred in the form of an error to the values of  $RI$  obtained by the majority of authors.

For nuclei having a fission threshold, such as  $^{231}\text{Pa}$ ,  $^{243}\text{Am}$  and  $^{249}\text{Bk}$ , such an approach is erroneous. Here  $RI_f$  is a function strictly dependent on the upper integration limit and consists of two principal parts.

(1) A low-energy part from 0.5 eV to several keV and (2) from a threshold neutron energy value to infinity. The intermediate energy region from several keV to the fission threshold contributes a negligible amount of the order of 0.1 barn to  $RI_f$ .

Calculation of  $RI_f$  for  $^{243}\text{Am}$  from the differential data in Ref. [27] showed that an increase of  $E_n$  from 10 to 20 MeV increases  $RI_f$  correspondingly from 3.8 to 5.4b, i.e. almost by a factor of 1.5. From experiments it follows that for the same  $^{243}\text{Am}$  the value of  $E_k$  fluctuates from  $\sim 30$  keV [11] to 20 MeV [27]. Thus, for isotopes with a fission threshold substitution of  $E_k$  in the upper part of the neutron spectrum and scatter in the value of  $E_k$  can be a cause of discrepancies in  $RI_f$ .

When  $RI_f$  is obtained directly in integral measurements in reactors, it is also necessary to allow for the perturbation of the relationship  $1/E$  in the fission neutron spectrum above 100 keV (here the spectrum assumes an exponential dependence). Isotopes having a fission threshold are particularly sensitive to this perturbation. Therefore it is imperative that a correction be made to  $RI_f$  for the discrepancy between the actual neutron spectrum and the  $1/E$  spectrum. As a result, scatter may arise in the values of the resonance integrals due to

significant differences in the neutron energy spectra of different reactors and due to the thickness and shape of the cadmium filters.

Review of publications on resonance integrals and thermal cross-sections

Resonance integrals calculated from resonance parameters as well as measured cross-sections for energy 0.0253 eV have a very high accuracy. Unfortunately, there is very little such data [10, 16-19, 25, 27, 28, 37, 38, 41, 47, 51, 68, 78, 84].

Direct measurements by the "cadmium difference" method on a Maxwellian spectrum with sufficiently pure samples have been performed in Refs [1, 2, 11, 12, 14, 15, 20, 21, 24, 26, 39, 40, 43-46, 50, 53, 54, 65, 69-73, 75, 77]:

Let us now consider the data obtained by the isotope accumulation method [4-9, 13, 22, 23, 26, 29, 30, 35, 36, 42, 45, 49, 60-64, 66, 67, 74, 83, 85]. The sample being investigated, shielded with a cadmium foil, is irradiated in the reactor. The quantity of isotopes accumulating in it is determined on a mass spectrometer after the actinides (elements) have been chemically separated. Then nuclear data taken from the literature are fitted with the aid of the computer so as to reproduce the experimentally found isotopic composition of the irradiated sample. The ratio of the thermal to the epithermal flux (Cd-ratio) is also varied, in order to obtain optimum correspondence between experiment and calculation. The values of  $\sigma$  and RI are the quantities fitted to experiment in these measurements.

However, the isotopes accumulating in the sample appear as result of multiple neutron absorption in competition with different forms of decay of the intermediate nuclei. Matching of the isotope decay scheme and the corresponding self-consistent set of nuclear constants with the quantity of accumulated isotopes requires very precise knowledge of these constants for all the intermediate processes which is virtually unavailable at the present time. It

follows therefore that the accuracy of the accumulation method is low and the data obtained, particularly the values of RI, should be treated with caution.

In many publications there are data on total- $RI_{tot}$ , and absorption resonance integrals,  $RI_{\alpha}$ , as well as on neutron absorption cross-sections,  $\sigma_{\alpha}$ . When there is a lack of data on capture, it is possible to use the quantity  $\sigma_{\alpha} = \sigma(n,\gamma) + \sigma(n,f)$ , if in this case  $\sigma(n,f) \ll \sigma(n,\gamma)$ , and to consider as a first approximation that  $\sigma_{\alpha} \approx \sigma(n,\gamma)$  [6, 23, 26, 29, 30, 35, 44, 48, 61, 70]. If for the resonance integrals the condition  $RI_f \ll RI_{\gamma}$  is also fulfilled, it can be assumed as a first approximation that  $RI_{\gamma} = RI_{\alpha} = RI_{tot}$ . Thus, for even isotopes of curium  $RI_{\gamma} < RI_f$  by a factor of 10-50, for  $^{241}\text{Am}$  by a factor of 100-150, and for  $^{243}\text{Am}$  by a factor of 200-300. This circumstance makes it possible to employ for  $RI_{\gamma}$  the values of  $RI_{\alpha}$  and  $RI_{tot}$  obtained in Refs [5, 6, 22-25, 28-30, 38, 51].

To obtain absolute values of cross-sections measured by relative methods, reference cross-sections (standards) of such reactions as  $^{59}\text{Co}(n,\gamma)$ ,  $^{197}\text{Au}(n,\gamma)$ ,  $^{232}\text{Th}(n,\gamma)$ ,  $^{237}\text{Np}(n,\gamma)$ ,  $^{238}\text{U}(n,\gamma)$ ,  $^{235}\text{U}(n,f)$  and  $^{239}\text{Pu}(n,f)$  were used in Refs [1-16, 19-24, 29, 30, 33-36, 39-42, 46, 50, 52-57, 60, 62-67, 69-75]. In the course of time the standards have been changed and harmonized systems of reference cross-sections for these reactions have emerged. In the process of analysis part of the experimental data in Refs [3, 4, 7-10, 12, 13, 20, 24, 35, 39, 40, 57, 62-67, 72-74] have been subjected to renormalization, with the cross-sections and the resonance integrals from BNL-325 [31] being used as standards.

As a result of the analysis, the values of the cross-sections and the resonance integrals for certain isotopes have been shown to be invalid in a number of works.

1. Gavrilov, et al. [2, 46]. For  $^{241}\text{Am}$  the values of  $RI_{\gamma}$  and  $RI_f$  are too high due to uncertainty in the allowance for  $E_{cd}$ . Having checked the detector capabilities of the method

with the aid of  $^{239}\text{Pu}$  (first resonance at 0.296 eV), the authors evidently retained this value in all their measurements during recording.

Thus, the levels 0.307 eV of  $^{241}\text{Am}$  and 0.416 eV for  $^{243}\text{Am}$  (see Table 1) serve to make the values of  $\text{RI}_\gamma$  and  $\text{RI}_f$  too high; for  $^{241}\text{Am}$  they ought to be excluded from the evaluation. In the case of  $^{243}\text{Am}$  the level 0.416 eV is very weak, so that its contribution to  $\text{RI}_\gamma$  and  $\text{RI}_f$  may be neglected and these quantities themselves used in the evaluation.

Since the first levels for isotopes of Cm lie above 0.6 eV, the values obtained for the resonance integrals in this work are quite correct and suitable for evaluation.

2. Zhuravlev, et al. [11]. Here,  $\text{RI}_f$  of  $^{241}\text{Am}$  is almost two times higher than the  $\text{RI}_f$  obtained from differential data in Ref. [10]. Even if  $E_{cd}$  is reduced to the extremely low value of 0.3 eV, it is impossible to approximate  $\text{RI}_f$  for  $^{241}\text{Am}$  in Ref. [11] to the data in Ref. [10]. Evidently the calculation of the integrals contains some hidden error associated with  $E_{cd}$  resulting in the value of  $\text{RI}_f$  for  $^{241}\text{Am}$  being too high, so that it cannot be used in the evaluation. The data for  $^{243}\text{Am}$  and isotopes of Cm are quite suitable for evaluation.

3. Bak, et al. [1]. In this paper no information is provided on the "purity" of the samples and the value of  $E_{cd}$  (the thicknesses of cadmium are given). Analysis has shown that the authors had  $E_{cd} \sim 0.3$  eV, so that the lower levels of  $^{241}\text{Am}$  made a contribution to  $\text{RI}_\gamma$  and  $\text{RI}_f$ , and these values are left out of consideration.  $\text{RI}_\gamma$  and  $\text{RI}_f$  for  $^{243}\text{Am}$  may be used for evaluation in virtue of the fact that the contribution of the very weak level 0.416 eV is negligible.

4. Draper, et al. [14]. No information is provided on the value of  $E_{cd}$  so that the values of  $\text{RI}_f$  for americium-241, 242m and 243, being of illustrative character, are not used in the evaluation.

5. M. Darouzet, et al., [52].  $\sigma(n,\gamma)$  were determined in the neutron spectrum of the "Melusine" swimming-pool type reactor by the activation method. Analysis of the method indicates that the neutron spectrum deviates considerably from the Maxwellian, although a mean energy of 0.0253 eV was assigned to it, which excludes  $\sigma(n,\gamma)$  from consideration for americium-241 and 243 and curium-244. (The cross-sections themselves deviate from similar cross-sections in other works by 40-75%).

6. Schuman, et al., [5, 6, 29, 30]. Samples of Am and Cm were irradiated in cadmium cans 1, 3 and 7 mm thick but the numerical values of  $E_{cd}$  are not given and its effect on the experimental values of  $RI_\alpha$  is not analysed. Moreover, the isomer ratio of 0.773 found for  $^{241}\text{Am}$  [5] is much lower than the generally accepted value. Therefore the values of  $RI_\alpha$  for Am in Refs [5, 6, 30] are not used. The values of  $RI_\alpha$  for Cm isotopes in Refs [5, 29, 30] are quite suitable for evaluation, since the first levels of these isotopes are greater than  $E_{cd} = 0.5$  eV.

7. The values of  $RI_\gamma$  for  $^{231}\text{Pa}$  in publications [69, 72] are left out of consideration, since they were obtained for a cadmium cut-off energy from 0.1 to 0.3 eV and consequently were not shielded from the effect of levels 0.4 and 0.497 eV (see Table 1).

8. The values of  $RI_\gamma$  for  $^{231}\text{Pa}$  in Ref. [76], calculated from the resonance parameters for neutrons with energy from 0.1 to  $2 \cdot 10^3$  eV were not used, since a non-generally accepted lower boundary of  $E_{cd}$  was selected by the authors.

9. In Refs [61-66, 70, 83] an unjustifiably high accuracy of the cross-sections (from 3% [63] to 10% [65]), obtained by the isotope accumulation method, is observed. It is well known that this method can provide 15-30% accuracy of results, and the data of the above-mentioned works were in fact analysed with such accuracy.

10. Certain omissions and inaccuracies in the experiment on isotope accumulation in a graphite reactor as well as some confusion in the values of the standard -  $RI_{\alpha}$  for  $^{59}\text{Co}$  - made it impossible to use the results of Ref. [64].

11. The values of  $RI_f$  were measured incorrectly in Ref. [39]. The authors did not introduce a correction for scattering of neutrons by the hydrogen contained in the capsule with the curium and californium powders.

The thermal cross-sections and resonance integrals of the isotopes Pa, Am, Cm and Bk isotopes, selected as a result of analysis and appropriately classified, are listed in Table 2. The weighted mean values of these quantities that are recommended for use as evaluated values are also given in the table.

It should be noted that the limitations imposed on the use of  $RI_{\alpha}$  and  $RI_f$  do not extend to the thermal fission and capture cross-sections given in the same works [1, 2, 11, 14, 24].

## DISCUSSION OF THE RESULTS

### *Protactinium isotopes*

Evaluation of the neutron data of protactinium isotopes is complicated for the following reasons:

1. The process of neutron fission of  $^{231}\text{Pa}$  has been investigated very little, so that very little data is available on  $\sigma(n,f)$  [71, 80, 81], and no work at all has been published on  $RI_f$ ;
2. For  $^{233}\text{Pa}$  only evaluated values of  $\sigma(n,f)$  and  $RI_f$  have been published [31, 58]. Also it is not known whether this isotope has a fission threshold or is not fissioned at all by neutrons;

3. Where there is a dearth of neutron data,  $\sigma(n,\gamma)$  for energy 30 keV is used for the evaluation in the case of many isotopes. In the case of isotopes 231 and 233 such a possibility does not exist, because  $\sigma(n,\gamma)$  for them has not been measured at all and the values of  $\sigma_\gamma \sim \sigma_{\text{tot}}$  (for  $\sigma_\gamma \ll \sigma_f$ ), obtained from the total cross-sections, have been measured on a selector only up to 10 keV [68, 76].

### Protactinium-231

1. The mean values of all types of capture cross-sections agree with each other within the range  $\pm 10\%$ . Scatter in  $\sigma_\gamma(\text{eff})$  ranging up to 40% [69, 71, 74] is most often caused by the nature of the reactor neutron spectra.
2. It is evident that when  $E_{cd} = 0.4\text{-}0.414$  eV the strong level is shielded, and the contribution of the weak level to the value of  $RI_\gamma$  can be neglected as a first approximation and use made of the data in Refs [73, 78]. The fact that the values of  $RI_\gamma$  from Refs [73, 78] are  $\sim 30\%$  above the evaluated values of  $RI_\gamma^0$  [31, 58] cannot be explained simply by the contribution of the weak level because the value of  $RI_\gamma(\text{th}) = 775\text{b}$  [72], obtained for  $E_{cd} = 0.512$  eV is completely shielded from the effect of both levels, and it agrees well with  $RI_\gamma$  [73, 78]. A possible cause of  $RI_\gamma^0$  in Refs [31, 58] being too low could be the basic assumptions of the theory used in these evaluations.
3. The very low values of  $\sigma(n,f)$  compared with  $\sigma(n,\gamma)$  [71, 80, 81] are explained by the presence of a fission threshold at neutron energy  $\sim 400$  keV.
4. Values of  $RI_f$  for  $^{231}\text{Pa}$  have not been published.

### Protactinium-233

1. The capture of neutrons in  $^{233}\text{Pa}$  results in the formation of two isomers  $^{234g}\text{Pa}$  (6.75 h) and  $^{234m}\text{Pa}$  (1.18 min) for which the cross-sections have been obtained only by integral methods [65, 69]. The data in Refs [61, 64] are excluded (see our survey above).

For each of the isomers  $\sigma_{\gamma}(\text{th})$  [65] and  $\text{RI}_{\gamma}$  [65, 69] are in agreement with the analogous quantities of the evaluation in Ref. [31].

2. There is agreement of the total capture cross-sections in the Maxwellian neutron spectrum and disagreement by a factor of approximately 3.5 for  $\sigma_{\gamma}(\text{eff})$  obtained in reactor spectra [60, 66]. The fact that  $\sigma_{\gamma}(\text{eff})$  [60] agrees with the mean value of  $\sigma_{\gamma}(\text{th})$  allows  $\sigma_{\gamma}(\text{eff})$  [66] to be excluded from the analysis. Evidently the neutron spectrum of the MTR reactor [66] differs appreciably from the Maxwellian.

3. The total capture cross-section for isomeric states as well as the total capture integral practically agree with the total  $\sigma(n,\gamma)$  and  $\text{RI}_{\gamma}$  which were measured independently.

4. For the  $^{233}\text{Pa}$  (n,f) reaction there are no experimental data available.

### *Americium Isotopes*

#### Americium-241

1. Capture of neutrons in  $^{241}\text{Am}$  occurs with formation of two isomers -  $^{242}\text{Am}$  (16 h) and  $^{242m}\text{Am}$  (152 yr). There exist only integral experiments giving  $\sigma(\text{th})$  and  $\sigma(\text{eff})$ . For each isomer, agreement is observed between  $\sigma(\text{th})$  and  $\sigma(\text{eff})$  within the range 7-15%, but they differ by 23-43% from analogous cross-sections from BNL-325 [31].

The total capture cross-section of the isomeric states agrees well with the total capture cross-section measured independently.

All this leads to the conclusion that the values of  $\sigma(n,\gamma)$  for  $^{241}\text{Am}$  recommended in Ref. [31] are much too low.

2. The  $^{241}\text{Am}$  fission process was investigated using both differential and integral methods. The values of  $\sigma(n,f)$  obtained agree very well with each other and also with the evaluated values from Ref. [31].

3. Differential measurements of  $\text{RI}_\gamma$  do not exist. References [1, 2, 5] were excluded from consideration due to indeterminacy in the value of  $E_{cd}$ . As recommended values, it is proposed that  $\text{RI}_\gamma(\text{th})$  and  $\text{RI}_\gamma(\text{eff})$  from publications [3,8] be employed, since these agree with an accuracy of  $\pm 10\%$  and are in a good agreement (within the range 3-12%) with the data of Ref. [31].

4. The values of  $\text{RI}_f$  from Refs [1, 2, 11, 14] were excluded (see survey). The value of  $\text{RI}_f = 14.1\text{b}$  [10], calculated from data from differential measurements for the energy range 0.55 eV-18 MeV, is in good agreement with analogous values from the majority of evaluations [31, 32, 58]. However, there is a 33% disagreement with  $\text{RI}_f(\text{eff})$  in Ref. [8].

#### Americium-242g

1. In the only experimental work [13], performed in a reactor at Chalk River as far back as 1951 by the accumulation method, a value of  $\sigma(n,\gamma) = 5500\text{b}$  was obtained - after renormalization 5100b - which is recommended with 30% accuracy. A sole evaluation exists - JENDL-3 [58] - in which a non-renormalized cross-section from Ref. [13] is given as the recommended value.

2. Values of  $\sigma(n,f)$  have been measured only by integral methods [1, 4, 13, 21] and agree satisfactorily both with each other and with data of the evaluations in Refs [31] and [58].

3. There are no experimental data for  $RI_\gamma$  and  $RI_f$ , and evaluation of these quantities has been performed only in JENDL-3 [58].

#### Americium-242m

1. A value of  $\sigma(\text{eff}) = 1650\text{b}$  has been obtained solely in an experiment on isotope accumulation [4]. The evaluation of this quantity in BNL-325 [31] is acceptable.

2. The fission process was investigated both by differential and integral methods. From Table 2 it can be seen that all types of cross-sections agree well both with each other and with evaluated data from Ref. [31].

3. Experimental data for  $RI_\gamma$  do not exist, and evaluated values are given in four libraries with maximum scatter of 45% [32].

4. For  $RI_f$  the integral data in Refs [11, 14] are excluded. As result of analysis of the differential data in Refs [16, 17, 19] we recommend the value  $RI_f = 1677 \pm 85\text{b}$  which agrees (in the range 7-12%) with the same data in other libraries [31, 32, 58].

#### Americium-243

1. Values of  $\sigma(n,\gamma)$  have been measured only by integral methods and have 10% agreement. The values of  $\sigma(\text{th})$  and  $\sigma(\text{eff})$  put forward as recommended values agree very well with the data of the majority of evaluations [31, 32, 58].

2. In the case of  $^{243}\text{Am}$  the values of  $\sigma(n,f)$  were divided into two groups. The values for  $\sigma_f$  in Refs [2, 34] exceed those in Refs [11, 12, 33] by almost a factor of three. In Ref. [2], the error is great ( $\pm 55\%$ ) and it enables the fission cross-section to be used in a very wide range of dispersion of  $\sigma_f$  (from 90 to 310 mb). Moreover, the fission cross-section from Ref. [11] which is close to zero makes it possible to recommend a cross-section

$\sigma_f(\text{th}) = 74 \pm 4 \text{ mb}$  [33] (a 1989 work), which agrees with the evaluations of UKNDL-81 [32] and JENDL-3 [58].

3. For  $\text{RI}_\gamma$ , the data in Refs [1, 2] are excluded. The disparity between  $\text{RI}_\gamma^0 = 1825\text{b}$  [25], calculated from parameters for the neutron energy range 0.532 eV-350 eV and  $\text{RI}_\gamma(\text{eff}) = 2263\text{b}$  [22-24] amounts to 24%.

4. The values for  $\text{RI}_f$  in Refs [2, 14] are excluded. The value  $\text{RI}_\gamma^0 = 8.45\text{b}$  [27], calculated from differential data in the neutron energy range 0.5 eV-20 MeV, and the directly measured value  $\text{RI}_f(\text{th}) = 9.0\text{b}$  [11] agree with each other and with corresponding data of the evaluations in Refs [31, 32, 58].

### *Curium Isotopes*

#### Curium-242

1. There exists only a single experimental value for  $\sigma_\gamma(\text{eff}) = 25\text{b}$  [4], recalculation of which to the results  $\sigma_f = 5\text{b}$  and  $\text{RI}_f = 190\text{b}$  in Refs [57, 59] gives  $\sigma_\gamma(\text{eff}) = 20 \pm 5\text{b}$  (with allowance for the accuracy of the method), which agrees with the values for  $\sigma(n,\gamma)$  in other evaluations [31, 32, 58].

2. Only the effective fission cross-sections have been measured [4, 13], and they are cited in all evaluations [31, 32, 58].

3.  $\text{RI}_\gamma(\text{eff})$  has been measured in just one work on isotope accumulation [5]. In this situation,  $\text{RI}_\gamma^0$  may be replaced by  $\text{RI}_{\text{abs}}$  [38] obtained by calculation from the resonance parameters, since for  $^{242}\text{Cm}$   $\sigma_f < \sigma_g$  by a factor of 4-5.

4. In 1988, as a result of calculation from the resonance parameters, a value of  $RI_f = 12.9 \pm 0.7b$  was obtained [37]. Only two libraries have evaluated this quantity: JENDL-3 [58] and ENDL-82 [32], and their results exceed the value of  $RI_f^0$  in Ref. [37] by a factor of 1.5-3 respectively.

#### Curium-243

1. The values for  $\sigma(n,\gamma)$  and  $RI_\gamma$  are published in Ref. [54] and agree well with analogous data of BNL-325 [31].
2. Values for  $\sigma_f(th)$  and  $\sigma_f(eff)$  have been obtained in integral measurements, and the values recommended by us for these agree well with each other and with the data in Ref. [31].
3. Measurements of  $RI_f$  have been carried out only by integral methods and the scatter of the mean values amounts to only 8%.

#### Curium-244

The capture cross-sections have been obtained by integral methods and agree with each other within the limits of the above experimental errors. The absence of data from differential measurements of  $RI_\gamma$  is compensated by the value  $RI_{abs} = RI_\gamma^0$  calculated from the parameters [28], since for curium-244  $\sigma_f < \sigma_g$  by a factor of 10-15.

#### Curium-245

This isotope has been described in a large number of works with satisfactorily consistent data (see Table 2).

#### Curium-246

1.  $\sigma_{\gamma}(\text{eff})$  [35] is left out of account since it has a preliminary character and exceeds by an order of magnitude the remaining cross-sections which agree with each other within the limits of the above experimental errors.
2. The fission process has not been investigated very much but the values for  $\sigma_f$  and  $RI_f$  that do exist are in good agreement.

#### Curium-247

1. The scanty data on neutron capture is subject to great uncertainty (of the order of 50%). The value for  $\sigma_{\gamma}(\text{eff}) = \sigma_{\gamma}(\text{th}) = 60\text{b}$  measured in Refs [42, 46] is confirmed indirectly by the value  $\sigma_{\gamma}(\text{eff}) = 48\text{b}$  [22] obtained as the difference in the cross-sections  $\sigma_{\text{abs}} - \sigma_f$ , and is recommended by us for evaluation.
2. The available values of  $RI_{\gamma}(\text{th}) = 490 \pm 100$  [46] and  $RI_{\gamma}(\text{eff}) = 800 \pm 400$  [42] differ by 40% in absolute value but agree within the above error limits and do not contradict analogous data from evaluations [31, 32, 58].
3. The values of  $\sigma(n,f)$  measured in the Maxwellian spectrum decay in two groups, the cross-sections in which differ by 26-33%. On the basis of analysis of the integral experiments in Refs [11, 39, 40, 43] it is considered that the accuracy of  $\sigma_f(\text{th})$  should be from 5-10% to 20% and the normal mean fission cross-section taken as the recommended value.
4. The values of  $RI_f$ , which are presented in the form of preliminary values in Ref. [39] are left out of consideration. The recommended values for  $RI_f(\text{th})$  and  $RI_f(\text{eff})$  are not inconsistent with each other, although they differ in absolute value by 17%.

### Curium-248

Reference [46] is not taken into account owing to the indeterminacy in  $E_{cd}$ .

### Curium-249, 250

These two isotopes are investigated in passing mainly in accumulation experiments, and thus data is available only on the effective thermal capture cross-sections (see Table 2).

To date, only one library [58] has offered evaluated data for these curium isotopes, taking as a basis the values  $\sigma(n,\gamma)$  and  $\sigma(n,f)$  from Refs [55, 56].

It should be noted that in Ref. [59] the mean cross-sections  $\sigma(th)$  and  $\sigma(eff)$  for isotopes of curium are also evaluated separately.

### *Berkelium isotopes*

#### Berkelium-249

1. The fission process has hardly been investigated at all. The only experiment gives  $\sigma_f(eff) < 6$  barn [86]. Evaluated values of  $\sigma(n,f)$  and  $RI_f$  exist only in the libraries ENDL-82 [32] and JENDL-3 [58] and differ from each other by a factor of 2.5-3.
2. No direct measurements of  $\sigma^0(n,\gamma)$  exist. There are available integral data with large scatter of  $\sigma_\gamma(eff)$  and measurements of the total cross-sections at the thermal point, which enable us to evaluate the order of magnitude of  $\sigma^0(n,\gamma)$ . In fact, knowing  $\sigma_{tot}$ , measured by the time-of-flight method with good accuracy, and allowing for the relationship  $\sigma_f \ll \sigma_\gamma$  for  $^{249}\text{Bk}$ , we can consider as a first approximation that  $\sigma_{tot}^0 \approx \sigma^0(n,\gamma)$ . The value  $\langle \sigma^0(n,\gamma) \rangle = 658 \pm 90\text{b}$ , averaged from the data in Refs [82, 84], agrees with the value  $\sigma^0(n,\gamma) = 746\text{b}$  [84], calculated from the resonance parameters, and enables it to be stated that  $\sigma^0(n,\gamma)$  is located in the cross-section interval 650-750b.

An approximate value of  $\sigma_\gamma(\text{eff})$  can be derived from measurements on reactor spectra. According to the accuracy of the accumulation method, the values of  $\sigma_\gamma(\text{eff})$  in Refs [26, 83] are taken with a 30% error, and the value of the normal mean  $\langle \sigma_\gamma(\text{eff}) \rangle = 700 \pm 200\text{b}$  in Refs [26, 83, 85] is also included in the above capture cross-section interval.

3. Apart from measurements in two different kinds of experimental conditions [2], which gave consistent values of  $\text{RI}_\gamma(\text{th})$ , there are no other publications on resonance capture integrals. To confirm the data for  $\text{RI}_\gamma(\text{th})$  in Ref. [2], use is made of  $\text{RI}_{\text{tot}}^0$  calculated from the resonance parameters obtained from measurements of the total cross-sections for neutron energies below 46 eV [82] and below 100 eV [84]. Taking into account the relationship  $\sigma_\gamma \ll \sigma_\gamma$  for  $^{249}\text{Bk}$ , we can as a first approximation use  $\text{RI}_{\text{tot}}^0$  for  $\text{RI}_\gamma^0$  which agrees with  $\text{RI}_\gamma(\text{th})$  [2].

For  $^{249}\text{Bk}$  it is to be noted that the values of both  $\sigma(n,\gamma)$  and  $\text{RI}_\gamma$  from the libraries [31, 58] coincide, and this is confirmed by the corresponding cross-sections recommended in the present work

## REFERENCES

1. Bak, M.A., Krivokhvatskij, A.S., Peterzhak, K.A. et al., *At. Ehnerg.* 23 (1967) 316.
2. Gavrilov, V.D., Goncharov, V.A., Ivanenko, V.A. et al., *At. Ehnerg.* 41 (1976) 85.
3. Harbour R.M., MacMurdo K.W., McOrosson E.J.//*Nucl./sci.Eng.* 1973. V.50. P.364.
4. Inle H. Michael H. Neubert A. et al.//*J.In. Nucl. Chem.* 1972.V.34. P.2427.
5. Schuman R.P.//*Prog. WASH-1136.* 1969. P.53.
6. Deal R.A., Schuman R.P.//*Prog. WASH-1053.* 1964.P.76.
7. Hoff R.W., Hulet E.K., Michel M.C.//*Nucl. Eng.*1959. V.8.P.224.
8. Eberle S.H.//*Rep. KFK-1453.* 1971. P.1.
9. Eberle S.H.//*Rcp. KFK-1456.* 1971. P.51.
10. Dabbs J.W., Johnson C.H., Bemis C.E., et al.//*Nucl.Sci.Eng.* 1983.V.83. P.22.
11. Zhuravlev, K.D., Kroschkin, N.I., Chetverikov, A.G., *At. Ehnerg.* 39 4 (1975) 285.
12. Hulet E.K., Hoff R.W., Bowman H.R. et al.//*Phys. Rev.* 1957. V.107. P.1294.
13. Hanna G.C., Harvey B.G., Moss N.//*Phys.Rev.*1951. V.81. P.893.
14. Draper E.L., Jr.//*Nucl. Sci. Eng.* 1971. V.46. P.31.
15. Hyakutane W.//*Technology Report of The Kyushu Univ.* 1965. V.39. P.170. EXFOR-20274.004.
16. Browne J.C., White R.M., Howe R.E. et.al.//*Phys.Rev.* 1984. C-29. P.2188.
17. Bowman C.D., Auchampauch G.F., Fultz S.C. et al.//*Phys. Rev.* 1968. V.166. P.1219.
18. Perkin S.T., Auchampauch G.F. Hoff R.W. et.al.//*Nucl.Sci.Eng.* 1968. V.32. P.131.
19. Dabbs J.W., Johnson C.H., Bemis C.E. et.al.//*Nucl.Sci.Eng.* 1983. V.84. P.1.
20. Wolfsberg K., Ford G.P., Smith H.L.//*Nucl.Eng.* 1966. V.A/B-20. P.588.
21. Higgins G.H., Orande W.T.//*Phys. Rev.* 1951. V.94. P.735.
22. Smith J.A., Banick C.J., Folgar R.L.//*Proceed of 2-nd Conf. on Nuclear Cross Sections and Technology, Washington, March, 1968.* V. 2. P.1285.
23. Folgar R.L., Smith J.A., Browne C.D. et.al.// *Ibid*, V.2. P.1279.
24. Butler J., Lounsbury M., Merritt J.//*Can. J. Phys.* 1957. V.35. P.147.
25. Simpson O.D., Simpson F.B., Harvey J.A. et.al.//*Nucl.Sci.Eng.* 1974. V.55. P.273.
26. Ice C.H.//*Rep. DP-MS-66-69, [EXFOR-12550.001-.015].*
27. Knitter H., Budts-Jorgensen C.//*Nucl.Sci. Eng.* 1988. V.99. P.1.
28. Belanova, T.S., Zamyatin, Yu.S., Kolesov, A.G., *Proc. 3rd All-Union Sc. Conf. on Neutron Phys., Kiev, June 1975 (in Russian). Neutron Fiz., Moscow,* 3 (1976) 224.
29. Schuman R.P.//*Prog. WASH-1136.* 1969. p.54.
30. Schuman R.P.//*Prog. WASH-1124.* 1968. P.72.
31. Mughabghab S.F., Divadeenam M and Holden N.E.//*BNL-325 4-th edition, Neutron Cross Sections.* 1984. V.1. P. B.
32. *Compilation of Actinide Neutron Nuclear Data, Swedish nuclear data committee, Stockholm 1986.*
33. Wagemans C., Schillebeeckx P., Bocquet J.P.//*Nucl. Sci.Eng.* 1989. V.101. P.293.
34. Asghar M., Caitucoli F., Perkin P. et. al.//*Ann.Nucl.Eng.* 1979. V.6. P.661.
35. Stevens C.M., Studier M.N. Fields P.R. et.al.//*Phys. Rev.* 1954.V.94. P.974.
36. Fields P.R., Studier M.N., Stevens C.M. et.al.//*Phys.Rev.* 1956. V.120. P.180.
37. Alam B., Block R.C., Slovacek R.E. et. al.//*Nucl. Sci. ng.* 1988. V.99. P.268.

38. Artamonov, V.S., Ivanov, R.N., Kalebin, S.M. et al., Proc 4th All-Union Conf. on Neutron Phys., Kiev, April 1977 (in Russian). *Neutron Fiz.* 1 (1977) 257.
39. Halperin J., Oliver J.N., Stoughton R.W.//Rep. ORNL-4581. 1970. P.37.
40. Benjamin R.W., Macmurdo K.W., Spenser J.D.//*Nucl.Sci.Eng.* 1972. V.47. P.203.
41. Brown J.C., Benjamin R.W., Karraner D.G.//*Nucl.Sci.Eng.* 1978. V.65. P.166.
42. Thompson M.C., Hyder M.L., Reuland R.J.//*J.In.Nucl.Chem.-* 1971.-V.33.P.1553.
43. Diamond H., Hines J.J., Sjoblom et al.//*J.In Nucl. Chem.* 1968.V.30.P2553.
44. Jathey S.C.//*Nucl.Sci.Eng.* 1956. V.1. P.204.
45. Halperin J., Drushel R.E., Eby R.E.//*Prog. ORNL-4437.* 1969. P.20.
46. Gavrillov, V.D., Goncharov, V.A., *At. Ehnerg.* 44 (1968) 246.
47. Benjamin R.W., Ahlfeld C.E., Harvey J.A et.al.//*Nucl. Sci.Eng.* 1974. V.55. P.440.
48. Seaborg G.T., Private communication to Huizenga J.R. 1955 [EXFOR-12557. 1948].
49. Eastwood H.I.//*Nucl.Chem.* 1958. V.6. P.261.
50. Druschel R.E., Baybarz R.D., Halperin J.//*Prog. ORNL-4891.* 1973. P.23.
51. Berreth J.R., Simpson F.B., Rische B.C.//*Nucl.Sci.Eng.* 1972. V.49. P.145.
52. Darouzet M., Giacometti A., Giricud R., et al.//*Proceed. of Conf. on Nuclear data for Science and Technology, Antwerp, September 1982.* P.181.
53. Zhuravlev, K.D., Kroshkin, N.I., *At. Ehnerg.* 47 (1979) 55.
54. Bemis C.E., Jr. Oliver J.H., Eby R.//*Nucl.Sci. Eng.* 1977. V.63. P.413.
55. Loughheed R.W., Wild J., Hulet E.K., et al.//*J.In. Nucl.Chem.* 1978.
56. Diamond H., Hines J.J.//*Rep. ANL-7330.* 1967.
57. Benjamin R.W.//*Rep. IAEA-186.* 1976. V.2. P.46.
58. Shibata Kk. et al. R., JAERI-1319. (1990).
59. Belanova, T.S., *At. Ehnerg.* 53 6 (1982) 386.
60. Katzin L.I., Hasemann F. *Rep. OP. 3630.* 1946.
61. Katzin L.I. *Rep. ANL-WMM-1080.* 1953.
62. Connor J.C., Bayard R.T., MacDonald D. et al. *NSE.* 1967, v. 29.
63. Connor J.C. *rep. WAPD-TM-837,* 1970.
64. Smith R.R., Passel T.O., Reeber S.D. et al. *Rep. IDO-16226.* 1955.
65. Eastwood T.A., Warner R.D., *J.Can. Phys.,* 1960, v.38. p.751.
66. Halperin J., Stoughton R.W., Ellison C. *Nucl.Sci. Eng.,* 1956, v.1, p.1.
67. Halperin J., Stoughton R.W., Druscel R.E. *Rep. ORNL-3320,* 1962, p.1.
68. Simpson F.B., Coddling J.W. *NSE,* 1966, v.28, p.133.
69. Yurova, L.N., Polyakov, A.A., Ruchlo, V.P. et al., in: *Problems of Atomic Science and Technology, series: Nuclear Constants* 1 (1984) 55.
70. Smith R.R., Alley N.P., Lewis R.H. *PR,* 1956, v.101, p.1053.
71. Seaborg G.T., Manning N.M. *Rep. OS-3471,* 1846, p.2.
72. Gryntakis T.J. *Jn Nucl.* 1974, v.36, p.1447.
73. Aleksandrov, B.I., Bak, M.A., Krivokhvatskij, A.S., *At. Ehnerg.* 32 (1972) 178.
74. Elson R., Seller P.A., John E.R. *PR* (1953), v. 90, p.102.
75. Kobayashi K. *Prog. INDC(AP)-23L,* 1974, p.40.
76. Simpson F.B., Burgus W.N., Evans J.E. *NSE,* 1962, v.12, p.234.

77. Hashimoto S.I. Jour. Radioanalytical and Nucl. Chemistry, 1988, v.120, p.185.
78. Drane M.K., Nichols P.F. Rep. GA-7462, 1967.
79. Hilperin J., Bemis C.E. et al. R., ORNL-4706(1971).
80. Leonard H.H. Rep. Hw-67219, (1960).
81. Wagman C., D'Hondt P., Deruytter A.J. et al. Nucl. Phys. 1977. v. A-285, p.32.
82. Anufriev, V.A., Babich, S.I., Kocherygin, N.G. et al., At. Ehnerg. 55 5 (1983) 320.
83. Magnusson L.B., Studier M.H., Fields P.R. et al. Phys. Rev., 1954, v.96, p.1576.
84. Benjamin R.W., Harvey J.A., Hill N.W. et al. Nucl. Sci. Eng. 1983, v.95, p.581.
86. Loughheed R.W. Rep. WASH-1136, 1969, P.95.
87. Danon Y., R.E.Slovacek, R.C.Block et al. NSE-109, 34(1991).

Table 1

Characteristics of Pa, Am and Bk isotopes

Isotope	$E_n$ (eV)	$2gT_n$ (MeV)	$E_f$ (MeV)
$^{213}\text{Pa}$	0,400	0,0740	~0,4
	0,497	0,0134	
$^{241}\text{Am}$	0,307	0,0560	
	0,547	0,923	
$^{243}\text{Am}$	0,416	0,00084	0,7-0,8
$^{249}\text{Bk}$	0,195	0,102	1,07

Note:  $E_n$  is the position of the level,  $T_n$  is the neutron width of the level,  $E_f$  is the fission threshold

Table 2

Capture cross-sections in protactinium isotopes

Reaction	$\sigma_\gamma^0$ , barn	$\sigma_\gamma$ (th), barn	$\sigma_\gamma$ (eff), barn
$^{231}\text{Pa}(n, \gamma)$	200±5 [76] 260±13 [73]	200±5 [70]	175±30 [71]
		218±14 [72]	219±6 [69]
		201±22 [72]	299±60 [74] <sup>a)</sup>
		201±6 [75]	
		186±13 [77]	
weighted mean	208 19	201±13	218±13
$^{232}\text{Pa}(n, \gamma) ^{234}\text{Pa}$		25±6 [64] <sup>b)</sup>	32±8 [64] <sup>b)</sup>
		19±3 [65]	
		19±3 [65]	
$^{233}\text{Pa}(n, \gamma) ^{234m}\text{Pa}$		43±11 [64] <sup>b)</sup>	75±19 [64] <sup>b)</sup>
		20±4 [65]	
		20±4 [65]	
$^{233}\text{Pa}(n, \gamma) ^{234(m+g)}\text{Pa}$		39±4 [65]	37±14 [60]
		68±14 [64] <sup>b)</sup>	55±14 [61] <sup>b)</sup>
		42±5 [67]	107±27 [64] <sup>b)</sup>
		31,4±7,9 [63]	128±20 [66] <sup>b)</sup>
		39,0±3,5	37±14 [60]
weighted mean			

Note:

- (a) The data have been renormalized to the unified system of standards [31].
- (b) The data have been excluded from consideration for the reasons stated in the survey of works or in the process of analysis.
- (c) Owing to uncertainty in the value of  $E_{cd}$ , the data are used only as auxiliary values, particularly where information is lacking.
- (d) In brackets under the calculated values for  $RI_n$  and  $RI_f$  are shown the energy limits of integration given in the literature.
- (e) No data are available on  $RI_f$  for  $^{231}, ^{233}\text{Pa}$ .
- (f) The weighted mean values are recommended as evaluated values for the particular type of cross-section (resonance integral).
- (g) No data available on  $RI_f$  for  $^{249}\text{Bk}$ .

Table 2 (contd.)

Fission cross-section in Pa isotopes

Reaction	$\sigma_f^0$ , mbarn	$\sigma_f$ (th), mbarn	$\sigma_f$ (eff), mbarn
$^{231}\text{Pa}(n, f)$	14±25% [80]	19±4 [81]	10±5 [71]
weighted mean	14±4 [80]	19±4 [81]	10±5 [71]

Resonance capture integrals for Pa isotopes

Reaction	$RI_\gamma^0$ , barn	$RI_\gamma$ (th), barn	$RI_\gamma$ (eff), barn
$^{231}\text{Pa}(n, \gamma)$	760±80 [78]	775±70 [70] <sup>a)</sup>	
	(0 414 - ∞ eV)	800±80 [73] <sup>a)</sup>	
weighted mean	760±80 [78]	786±15	
$^{233}\text{Pa}(n, \gamma) ^{234g}\text{Pa}$		454±100 [65] <sup>a)</sup>	435±67 [69]
			487±122 [64] <sup>ab)</sup>
weighted mean		454±100 [65]	436±67 [69]
$^{233}\text{Pa}(n, \gamma) ^{234m}\text{Pa}$		464±90 [65] <sup>a)</sup>	722±180 [64] <sup>ab)</sup>
weighted mean		464±90 [65]	
$^{233}\text{Pa}(n, \gamma) ^{234(m+g)}\text{Pa}$	901±45 [68]	918±100 [65] <sup>a)</sup>	1209±302 [64] <sup>ab)</sup>
	(0, 4-10 <sup>4</sup> eV)	908±90 [67] <sup>a)</sup>	
		865±90 [63] <sup>a)</sup>	
weighted mean	901±45 [68]	897±23	

Table 2 (contd.)

Capture cross-sections in americium isotopes

Reaction	$\sigma_{\gamma}^0$ , barn	$\sigma_{\gamma}$ (th), barn	$\sigma_{\gamma}$ (eff), barn
$^{241}\text{Am}(n, \gamma)^{242g}\text{Am}$		670±60 [1]	775±80 [4]
		780±78 [2]	635±67 [7] <sup>a)</sup>
		748±20 [3]	
	weighted mean	745±24	693±69
$^{241}\text{Am}(n, \gamma)^{242m}\text{Am}$		70±5 [1]	95±26 [4]
		73±14 [2]	
		83,8±2,6 [3]	
	weighted mean	80±3	95±26 [4]
$^{241}\text{Am}(n, \gamma)^{242(m+g)}\text{Am}$		853±64 [2]	820±82 [6]
		740±65 [1]	
		831±23 [3]	
	weighted mean	824±20	820±32 [6]
$^{242g}\text{Am}(n, \gamma)$			5100±1550 [60] <sup>a)</sup>
$^{242m}\text{Am}(n, \gamma)$			1650±330 [4]
$^{243}\text{Am}(n, \gamma)$		73±6 [1]	78±6 [23]
		83±6 [2]	77±2 [9]
		72±2 [24] <sup>a)</sup>	70±15 [35] <sup>b)</sup>
		83±10 [26]	
	weighted mean	73±2	77±3

Table 2 (contd.)

Fission cross-sections in americium isotopes

Reaction	$\sigma_f^0$ , barn	$\sigma_f$ (th), barn	$\sigma_f$ (eff), barn
$^{241}\text{Am}(n, f)$	3, 13±0, 19[10]	a) 2, 80±0, 25[2]	2, 8±0, 3[13] <sup>a)</sup>
		3, 20±0, 15[11]	4, 0±0, 8[4]
		3, 15±0, 10[1]	
		2, 91±0, 14[12] <sup>a)</sup>	
		a) 3, 8±0, 2 [15]	
weighted mean	3, 13±0, 19[10]	3, 15±0, 13	3, 0±0, 3
$^{242g}\text{Am}(n, f)$		2950±800[21]	2100±420[4]
		2100±200[1]	2500±750[13]
	weighted mean	2150±200	2195±170
$^{242m}\text{Am}(n, f)$	6328±320[16]	6080±500[11]	6800±1360[4]
	6950±250[19]	7372±310[20] <sup>a)</sup>	
		5910±460[12] <sup>a)</sup>	
	weighted mean	6713±302	6748±488
$^{243}\text{Am}(n, f)$		0, 20±0, 11[2]	
		≤0, 067 [12] <sup>a)</sup>	
		0, 074±, 004[33]	
		~0, 0 [11]	
		. 1983±, 0042[34]	
	weighted mean		0, 074±0, 010

Table 2 (contd.)

Resonance capture integrals in americium isotopes

Reaction	$RI_{\gamma}^0$ , barn	$RI_{\gamma}$ (th), barn	$RI_{\gamma}$ (eff), barn
$^{241}\text{Am}(n, \gamma)^{242g}\text{Am}$		$2100 \pm 200 [1]^c)$ $1570 \pm 10 [2]^c)$	$1406 \pm 124 [3]^a)$
weighted mean		$1576 \pm 30^c)$	$1406 \pm 124 [3]$
$^{241}\text{Am}(n, \gamma)^{242m}\text{Am}$		$300 \pm 30 [1]^c)$ $230 \pm 80 [2]^c)$	$220 \pm 19 [3]^a)$
weighted mean		$291 \pm 23^c)$	$220 \pm 19 [3]$
$^{241}\text{Am}(n, \gamma)^{242(m, g)}\text{Am}$		$2400 \pm 200 [1]^c)$ $1800 \pm 100 [2]^c)$ $1626 \pm 143 [3]^a)$	$1100 \pm 100 [5]^c)$ $1469 \pm 147 [8]^a)$
weighted mean		$1626 \pm 143 [3]$	$1469 \pm 147 [8]$
$^{242g}\text{Am}(n, \gamma)$		no experimental data	
$^{242m}\text{Am}(n, \gamma)$		no experimental data	
$^{243}\text{Am}(n, \gamma)$	$1825 \pm 80 [25]$ ( $E_{\kappa} = 250$ eV)	$2300 \pm 200 [1]^c)$ $2210 \pm 150 [2]^c)$	$2290 \pm 50 [22]$ $2250 \pm 50 [23]$ $2249 \pm 50 [24]^a)$ $1930 \pm 190 [9]$
weighted mean	$1825 \pm 80 [25]$	$2257 \pm 46^c)$	$2250 \pm 20$

Table 2 (contd.)

Resonance fission integrals in americium isotopes

Reaction	$RI_f^o$ , barn	$RI_f$ (th), barn	$RI_f$ (eff), barn
$^{241}\text{Am}(n, f)$	$14, 1 \pm 0, 9 [10]$ ( $E_k=1$ MeV)	$22, 5 \pm 1, 7 [2]^c)$ $21 \pm 2 [1]^c)$ $27, 7 \pm 1, 6 [11]^c)$	$22, 2 \pm 2, 1 [8]^a)$
weighted mean	$14, 1 \pm 0, 9 [10]$	$21, 9 \pm 1, 7$	$22, 2 \pm 2, 1 [8]$
$^{242g}\text{Am}(n, f)$	no experimental data		
$^{242m}\text{Am}(n, f)$	$1553 \pm 78 [16]$ ( $E_k=20$ MeV)	$2260 \pm 200 [11]^c)$	
	$1570 \pm 110 [17]$ ( $E_k=3.7$ MeV)	$2027 \pm 100 [14]^c)$	
	$1800 \pm 65 [19]$ ( $E_k=20$ MeV)		
weighted mean	$1677 \pm 85$	$2070 \pm 100$	
$^{243}\text{Am}(n, f)$	$8, 45 \pm 0, 45 [27]$ $E_k=2$ MeV)	$17, 1 \pm 1, 3 [2]^c)$ $9, 0 \pm 1, 0 [11]$	
weighted mean	$8, 45 \pm 0, 45 [27]$	$9, 0 \pm 1, 0 [11]$	

Table 2 (contd )

Capture cross-sections in curium isotopes

Reaction	$\sigma_{\gamma}^0$ , barn	$\sigma_{\gamma}$ (th), barn	$\sigma_{\gamma}$ (eff), barn
$^{242}\text{Cm}(n, \gamma)$			$20 \pm 5 [4, 57]^a)$
$^{243}\text{Cm}(n, \gamma)$		$130, 7 \pm 9, 6 [54]$	$137, 4 \pm 9, 6 [54]$ $200 \pm 100 [4]$
weighted mean		$130, 7 \pm 9, 6 [54]$	$137, 4 \pm 9, 6 [54]$
$^{244}\text{Cm}(n, \gamma)$		$15, 2 \pm 1, 2 [46]$	$25 \pm 10 [35]$ $14 \pm 4 [42]$ $8, 4 \pm 4, 2 [23]$ $14, 5 \pm 4 [22]$
weighted mean		$15, 2 \pm 1, 2 [46]$	$13, 1 \pm 2, 2$
$^{245}\text{Cm}(n, \gamma)$	$341 \pm 34 [41]$		$340 \pm 20 [45]$ $200 \pm 100 [35]$ $360 \pm 50 [42]$ $260 \pm 130 [26]$
weighted mean	$341 \pm 34 [41]$		$336 \pm 16$
$^{246}\text{Cm}(n, \gamma)$		$1, 14 \pm 0, 30 [46]$ $1, 2 \pm 0, 4 [45]$	$1, 5 \pm 0, 5 [42]$ $15 \pm 10 [35]^b)$
weighted mean		$1, 16 \pm 0, 03$	$1, 5 \pm 0, 5 [42]$
$^{247}\text{Cm}(n, \gamma)$		$60 \pm 20 [46]$	$60 \pm 30 [42]$ $48 [22]^b)$
weighted mean		$60 \pm 20 [46]$	$60 \pm 30 [42]$
$^{248}\text{Cm}(n, \gamma)$	$2, 59 \pm 0, 26 [47]$	$10, 7 \pm 1, 5 [46]^b)$ $2, 63 \pm 0, 26 [50]$	$3 \pm 1 [42]$ $6, 0 \pm 2, 5 [49]$
weighted mean	$2, 59 \pm 0, 26 [47]$	$2, 63 \pm 0, 26 [50]$	$3, 4 \pm 1, 0$
$^{249}\text{Cm}(n, \gamma)$			$1, 6 \pm 0, 8 [56]$
$^{250}\text{Cm}(n, \gamma)$			$80 \pm 30 [55]$

Table 2 (contd.)

Fission cross-sections in curium isotopes

Reaction	$\sigma_f^o$ , barn	$\sigma_f$ (th), barn	$\sigma_f$ (eff), barn
$^{242}\text{Cm}(n, f)$			4,6 [4, 13] <sup>a)</sup>
$^{243}\text{Cm}(n, f)$		579±31 [54] 640±46 [12] <sup>a)</sup> 672±60 [53]	633±27 [54] 750±75 [4]
weighted mean		609±26	646±37
$^{244}\text{Cm}(n, f)$		1,0±0,2[11] 1,14±0,5[40] <sup>a)</sup>	1,5±10[42]
weighted mean		1,02±0,05	1,5±1,0[42]
$^{245}\text{Cm}(n, f)$	2143 58 [41]	1900±100[2] 2070±150[11] 1740±140[12] <sup>a)</sup> 1920±180[39] 2088±37[40] <sup>a)</sup> 2040±80[43]	1880±300 [23] 2000±150 [35] 1800±300 [36] 2030±200 [42] 2000±150 [26]
weighted mean	2143 58 [41]	2043±38	1980±32
$^{246}\text{Cm}(n, f)$		0,14±0,05[11] 0,176±0,005[40] <sup>a)</sup>	
weighted mean		0,158±0,080	
$^{247}\text{Cm}(n, f)$	106±53 [87]	80±7 [11] 85±5 [40] <sup>a)</sup> 108±5 [43] 120±12 [39]	100±50 [42]
weighted mean	106±53 [87]	98,3±19,7	100±50 [42]
$^{248}\text{Cm}(n, f)$		0,35±0,07[40] <sup>a)</sup> 0,39±0,08[11]	
weighted mean		0,37±0,04	

Table 2 (contd.)

Resonance capture integrals for curium isotopes

Reaction	$RI_{\gamma}^0$ , barn	$RI_{\gamma}(\text{th})$ , barn	$RI_{\gamma}(\text{eff})$ , barn
$^{242}\text{Cm}(n, \gamma)$	115±53 [38]		150±40 [5]
$^{243}\text{Cm}(n, \gamma)$			214, 4±20, 3[54]
$^{244}\text{Cm}(n, \gamma)$	643±53 [28]	626±53 [46]	650±50 [42] 650±50 [23]
weighted mean	643±53 [28]	626±53 [46]	650±25
$^{245}\text{Cm}(n, \gamma)$	100 [41]		110±20 [42] 101±8 [45]
weighted mean	100 [41]		102±3
$^{246}\text{Cm}(n, \gamma)$	101±11 [47] ( $E_{\text{c}}=5$ keV)	118±15 [46] 121±7 [45]	135±25 [42] 110±40 [29]
weighted mean	101±11 [47]	120±1	128±11
$^{247}\text{Cm}(n, \gamma)$		490±100[46]	800±400[42]
$^{248}\text{Cm}(n, \gamma)$	259±12 [47] ( $E_{\text{c}}=3$ keV)	250±24 [46] 267±27 [50]	275±75 [42]
weighted mean	259±12 [47]	275, 5±8, 5	275±75 [42]

Table 2 (contd.)

Resonance fission integrals for curium isotopes

Reaction	$RI_f^0$ , barn	$RI_f(th)$ , barn	$RI_f(eff)$ , barn
$^{242}\text{Cm}(n, f)$	$12,9 \pm 0,7 [37]$ ( $E_k=50,9$ keV)		
$^{243}\text{Cm}(n, f)$		$1480 \pm 150 [53]$	$1575 \pm 136 [54]$ $1860 \pm 400 [42]$
weighted mean		$1480 \pm 150 [53]$	$606 \pm 87$
$^{244}\text{Cm}(n, f)$		$13,4 \pm 1,5 [11]$ $18,0 \pm 3,6 [40]^a)$	$12,5 \pm 2,5 [42]$
weighted mean		$14,1 \pm 1,4$	$12,5 \pm 2,5 [42]$
$^{245}\text{Cm}(n, f)$	$721 \pm 20\% [41]$	$850 \pm 60 [2]$ $805 \pm 80 [11]$ $1124 \pm 100 [39]^a)$ $789 \pm 40 [40]^a)$	$750 \pm 50 [23]$
weighted mean	$721 \pm 20\% [41]$	$835 \pm 53$	$750 \pm 50$
$^{246}\text{Cm}(n, f)$		$13,3 \pm 1,5 [11]$ $10,2 \pm 0,4 [40]^a)$	
weighted mean		$10,4 \pm 0,8$	
$^{247}\text{Cm}(n, f)$	$890 \pm 53 [87]$ ( $0,1\text{eV}$ $80\text{keV}$ )	$730 \pm 70 [11]$ $798 \pm 50 [40]^a)$ $1045 \pm 110 [39]^ab)$	$935 \pm 190 [42]$
weighted mean	$890 \pm 53 [87]$	$775 \pm 32$	$935 \pm 190 [42]$
$^{248}\text{Cm}(n, f)$		$13,6 \pm 0,8 [40]^a)$ $13,1 \pm 1,5 [11]$	
weighted mean		$13,6 \pm 0,1$	

Table 2 (contd.)

Capture cross-sections in berkelium isotopes

Reaction	$\sigma_{\gamma}^0$ , barn	$\sigma_{\gamma}$ (th), barn	$\sigma_{\gamma}$ (eff), barn
$^{249}\text{Bk}(n, \gamma)$	710±40 [84]	1800±10 [2] <sup>b)</sup>	1100±300 [85]
	500±70 [82]		660±200 [26]
			350±150 [83]
weighted mean	658±90		700±200

Fission cross-sections in berkelium isotopes

Reaction	$\sigma_f^0$ barn	$\sigma_f$ (th) barn	$\sigma_f$ (eff) barn
$^{249}\text{Bk}(n, f)$		<6 [86]	

Resonance capture integral in berkelium isotopes

Reaction	$RI_{\gamma}^0$ , barn	$RI_{\gamma}$ (th), barn	$RI_{\gamma}$ (eff), barn
$^{249}\text{Bk}(n, \gamma)$	870±120 [82] (0,4+46 eV)	1100±100 [2]	
	1087±110 [84] (0,625±100 eV)	1300±300 [2]	
weighted mean	988±108	1120±80 [2]	

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