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**Mechanical Impacts of Poloidal Eddy Currents  
on the Continuous Vacuum Vessel of a Tokamak**

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**Korea Atomic Energy Research Institute**

## 제출문

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본 보고서를

**Mechanical Impacts of Poloidal Eddy Currents  
on the Continuous Vacuum Vessel of a Tokamak**

에 관한 기술보고서로 제출합니다.

1996 년 11 월

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## ABSTRACT

Poloidal eddy currents are induced on the continuous torus vacuum vessel by changes of the toroidal field during the machine start-up(toroidal field coil charge), shut-down(toroidal field coil discharge) and plasma disruption(plasma diamagnetism change). Analytic forms for the eddy currents flowing on the vessel, consequent pressures and forces acting on it are presented in this report. The results are applied to typical operation modes of the KT-2 tokamak.

Stress analysis for two typical operation modes of toroidal field damping during a machine shut-down and plasma energy quench during a plasma disruption were carried out using 3D FEM code(ANSYS 5.2).

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# Mechanical impacts of poloidal eddy currents on the continuous vacuum vessel of a tokamak

## Abstract

Poloidal eddy currents are induced on the continuous torus vacuum vessel by changes of the toroidal field during the machine start-up(toroidal field coil charge), shut-down(toroidal field coil discharge) and plasma disruption(plasma diamagnetism change). Analytic forms for the eddy currents flowing on the vessel, consequent pressures and forces acting on it are presented in this report. The results are applied to typical operation modes of the KT-2 tokamak.

Stress analysis for two typical operation modes of toroidal field damping during a machine shut-down and plasma energy quench during a plasma disruption were carried out using 3D FEM code(ANSYS 5.2).

## I. Introduction

One of the critical considerations in tokamak design for mechanical engineers, electrical engineers and plasma physicists is the effect of eddy currents induced on the vacuum vessel. Vessel eddy currents are induced due to transient magnetic fields in practically every tokamak operation stage. The eddy currents may give rise to damaging forces in the vessel structure[1,2,3].

During plasma initiation, vertical displacement and plasma current quench, variations of poloidal magnetic fields produce toroidal vessel currents, while poloidal vessel currents are generated due to changes of the toroidal magnetic field in the stages of TF(toroidal field) rising and TF damping. The change of poloidal beta and consequent variation of plasma diamagnetism in a plasma disruption also produces poloidal eddy currents on the vessel. The vessel currents interact with magnetic fields orthogonal to them, and the resultant electromagnetic load is perpendicular to the vessel surface. In addition to the mechanical impacts the eddy currents also inhibit plasma formation, equilibrium field ramp-up and plasma active control. They, however, help fast passive control of the plasma and shield magnets placed outside the vessel from electromagnetic transient. The practical level of the effect of the vessel eddy current depends on the relative magnitude of the magnetic field diffusion(or current decay) time constant compared with the change rate of the applied field. The longer

the diffusion time is, the stronger the effect is.

This report describes the procedure of estimating the effect of vessel poloidal eddy currents produced by the changes of the toroidal field due to any causes. In Sec. II general expressions for the eddy currents induced on the all-welded torus vacuum vessel and pressures acting on the vessel when the toroidal field coil is charged or discharged. The mode of the diamagnetic change in a plasma disruption is considered in Sec. III. Application of the results on the KT-2 tokamak is found in each section. Finally, the results of the stress analysis in TF damping and plasma energy quench as typical operation modes producing poloidal eddy currents are discussed in Sec. IV.

## II. Charging and discharging the toroidal field coil[4]

Figure 1 shows a toroidal field coil modeled as a current sheet and a conducting torus shell located inside the coil. The coil and the vessel have arbitrary cross-sections.

Before start-up( $t < 0$ ) the field is zero on both sides of the vessel, while the coil field  $B$  exists in the steady state with the toroidal field coil charged fully. The field inside the vessel starts to increase or decrease as the coil is charged or discharged. There may be delays in feeling the field change inside the vessel due to the finite field diffusion time depending on the vessel material and geometry. The toroidal field is usually varied so slowly that it may be assumed to be a quas-steady one. That is, the field inside the vessel is just the same field applied by the toroidal field coil, without any delay and any distortion, except minor contribution from vessel currents.

The locally induced magnetic pressure is given by Equation (1).

$$P = J_v \left( B_t + \frac{B_i}{2} \right) t_v \quad (1)$$

where,  $P$ : magnetic pressure acting on the vessel at a observing point(= $P(r)$ )[Pa]

$J_v$ : current density flowing on the vessel(= $J_v(r)$ )[A/m<sup>2</sup>]

$B_t$ : magnetic field applied by toroidal field coil(= $B_t(r)$ )[T]

$B_i$ : magnetic field induced by the vessel eddy currents(= $B_i(r)$ )[T], 1/2 is introduced because the poloidal current does not produce toroidal field outside the current shell, then in the vessel wall  $B_i$  is assumed to be changed linearly from zero at the outer surface to the full value at the inner surface



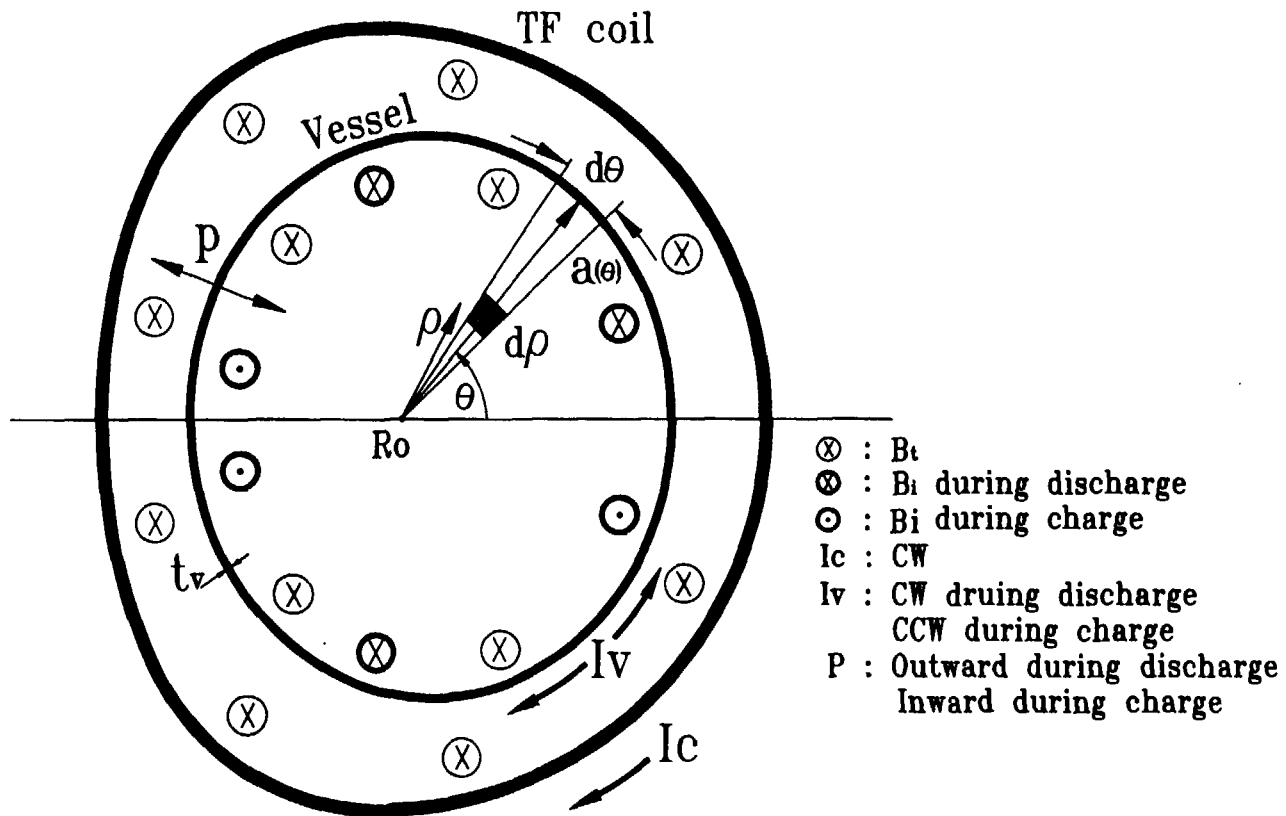


Fig.1. Current and pressure induced on a vessel placed within a toroidal field coil.

$t_v$ : thickness of the vessel wall[m]

$r$ : major radius to the point in question[m].

The pressure is composed of two terms( $P=P_0+P_i$ ), one from the applied field( $P_0=J_v B_i t_v$ ) and the other from the induced field( $P_i=J_v B_i t_v/2$ ). The direction of the pressure depends on the directions of the vessel current and the magnetic field. The direction of  $B_i$  is not changed in spite of variation in the magnitude, and that of induced current depends on the sign of change rate of  $B_i$ . The first term has minus sign(inward) when the field decreases, and plus(outward) when the field increases. The second term has always plus because the induced field has the same sign as the vessel current. The magnitude of the second term is expected to be much lower than the first term because it is second-order.

The power supply of the toroidal field coil may be considered as a constant voltage source. Because the coil is a inductive load the coil current and the field varies in accordance with a exponential function. To shorten the time taken to reach the flat top power supply voltage is increased a little higher(a few tens percent) than a steady one at the flat top. The ratio of the two volages is called overvoltage or forcing factor and is denoted usually by  $\alpha$ . If the toroidal field coil is charged from zero according to Equation (2), we can obtain expressions of  $J_v$ ,  $B_i$  and consequently  $P$  in Equation (1) step by step.

$$B_i = \alpha B_{i0} \frac{R_p}{r} (1 - e^{-t/\tau}) \quad (2)$$

where,  $\alpha$ : overvoltage factor(=supplied voltage/flat-top voltage)

$B_{i0}$ : flat-top mafnetic field at the plasma center[T]

$R_p$ : major radius to the plasma center[T]

$\tau$ : time constant of the coil(= $L_c/R_c$ ,  $L_c$  is the coil inductance and  $R_c$  is the coil resistance)[sec]

$t$ : time[sec]

The induced vessel poloidal current is governed by the circuit equation as follows,

$$L_v \frac{dI_v}{dt} + R_v I_v = - \frac{d\Phi}{dt} \quad (3)$$

where,  $L_v$ : poloidal inductance of the vessel[H]

$R_v$ : poloidal resistance[ $\Omega$ ]

$I_v$ : vessel poloidal current(= $2\pi r t_v J_v(r)$ )[A]

$\Phi$ : magnetic flux linking the cross-section of the vessel[Wb].

$\Phi$  is obtained by integrating applied magnetic field over the cross-sectional area of the vessel and is given by Equation (4)(refer to Fig. 1 for notations of the variables used in the equation).

$$\begin{aligned}\Phi &= \int B_t dA = \alpha B_0 R_p (1 - e^{-U\tau}) \int \frac{dA}{r} \\ &= \alpha B_0 R_p (1 - e^{-U\tau}) \int_0^{2\pi} \int_0^a \frac{\rho d\rho d\theta}{R_0 + \rho \cos\theta} = CF(1 - e^{-U\tau})\end{aligned}\quad (4)$$

where, a: minor radius to the vessel wall(= $a(\theta)$ )[m]

$\rho$ : minor radius to the infinitesimal element of surface integration[m]

$R_0$ : major radius to the vessel center[m]

C: constant(= $\alpha B_0 R_p$ )

F: double integral term.

$L_v$  and  $R_v$  are calculated by the vessel geometry and are expressed by Equations (5) and (6) respectively.

$$L_v = \frac{d\Phi_v}{dI_v} = \frac{\mu_0}{2\pi} F \quad (5)$$

where,  $\Phi_v$ : flux linking the vessel cross-section, generated by the vessel current[Wb]

$\mu_0$ : permeability in vacuum.

$$R_v = \int \frac{\eta d\ell}{2\pi r t_v} = \frac{\eta}{2\pi t_v} \int_0^{2\pi} \frac{\sqrt{(a^2 d\theta^2 + da^2)}}{R_0 + a \cos\theta} \approx \frac{\eta}{2\pi t_v} \int_0^{2\pi} \frac{ad\theta}{R_0 + a \cos\theta} = \frac{\eta}{2\pi t_v} G \quad (6)$$

where,  $\eta$ : resistivity of the vessel material[ $\Omega.m$ ]

G: integral term.

The induced vessel current  $I_v$  is obtained by inserting Equation (4) into Equation (3), and by solving it.  $I_v$  is given by Equation (7).

$$I_v = - \frac{\alpha B_0 R_p F}{\tau R_v (1 - \frac{\tau_v}{\tau})} (e^{-\nu \tau} - e^{-\nu \tau v}) = - \frac{2 \pi \alpha \sigma B_0 R_p t_v}{\tau (1 - \frac{\tau_v}{\tau})} \frac{F}{G} (e^{-\nu \tau} - e^{-\nu \tau v}) \quad (7)$$

where, - sign represents the current is induced to oppose the field increase

$\sigma$ : conductivity of the vessel material(=1/η)[mho/m]

$\tau_v$ : time constant of the vessel as a poloidal circuit(=L<sub>v</sub>/R<sub>v</sub>=σ μ<sub>0</sub>t<sub>v</sub>F/G)[sec].

Using relations J<sub>v</sub>=I<sub>v</sub>/2πr<sub>v</sub> and B<sub>i</sub>=μ<sub>0</sub>I<sub>v</sub>/2πr, and inserting Equations (2) and (7) into Equation (1), Equation (8) for the pressure acting on the vessel is obtained.

$$P = - \frac{\alpha^2 B_0^2 R_p^2 \sigma t_v}{r^2 \tau (1 - \tau_v/\tau)} \frac{F}{G} (1 - e^{-\nu \tau} - \frac{\mu_0 \sigma t_v}{2 \tau (1 - \tau_v/\tau)} \frac{F}{G} (e^{-\nu \tau} - e^{-\nu \tau v})) (e^{-\nu \tau} - e^{-\nu \tau v}) \quad (8)$$

where, - sign indicates inward pressure on the vessel wall.

If the vessel has a circular cross-section(a(θ)=a<sub>v</sub>), F and G are solved analytically; F(circular)=2π(R<sub>0</sub>-√(R<sub>0</sub><sup>2</sup>-a<sub>v</sub><sup>2</sup>)), G(circular)=2πa<sub>v</sub>√(R<sub>0</sub><sup>2</sup>-a<sub>v</sub><sup>2</sup>), and F/G=a<sub>v</sub>D(D is a constant, =1+A√(A<sup>2</sup>-1)-A<sup>2</sup>, A is the aspect ratio of the vessel(=R<sub>0</sub>/a<sub>v</sub>), D→1 when A>2). From Equations (5) and (6) an intuitive expression of τ<sub>v</sub> for circular cross-section is obtained as Equation (9).

$$\tau_v(\text{circular}) = \mu_0 \sigma t_v a_v D \quad (9)$$

The ratio τ<sub>v</sub>/τ (equivalent to, so called, magnetic Reynolds number R<sub>m</sub>) is negligible when the field change is so slow that the relation τ >> μ<sub>0</sub>σt<sub>v</sub>a is satisfied even in non-circular cross-sections. Then Equations (7) and (8) may be changed to approximate forms (10) and (11) respectively.

$$I_v = - \frac{2 \pi \alpha \sigma t_v B_0 R_p}{\tau (1 - \tau_v/\tau)} \frac{F}{G} e^{-\nu \tau} \quad (10)$$

$$\approx - \frac{2 \pi \alpha \sigma t_v B_0 R_p}{\tau} \frac{F}{G} e^{-\nu \tau} \quad (10)'$$

$$P = - \frac{\alpha^2 B_0^2 R_p^2 \sigma t_v}{r^2 \tau (1 - \tau_v/\tau)} \frac{F}{G} (1 - (1 + \frac{\mu_0 \sigma t_v}{2 \tau (1 - \tau_v/\tau)} \frac{F}{G}) e^{-\nu \tau}) e^{-\nu \tau} \quad (11)$$

$$\approx - \frac{\alpha^2 B_0^2 R_p^2 \sigma t_v}{r^2 \tau} \frac{F}{G} \left( 1 - \left( 1 + \frac{\mu_0 \sigma t_v}{2 \tau} \frac{F}{G} \right) e^{-\nu \tau} \right) e^{-\nu \tau} \quad (11)'$$

The pressure calculated by Equation (11)' is minus for the period of  $0 \leq t < \tau \ln(1+R_m/2) \approx \tau \sqrt{2}$  (maximum error is  $-\alpha^2 R_p^2 R_m^2 B_0^2 / 2 \mu_0 r^2$  at  $t=+0$ ), where  $R_m$  is defined as  $R_m = \tau \sqrt{2} = \sigma \mu_0 t_v F / \tau G$ , and final error at  $t \gg 0$  is about  $R_m \times 100\%$  arising from deletion of the term  $(1 - \tau / \tau \nu)$ . The pressure has a maximum of  $\alpha^2 R_p^2 (R_m / (2 + R_m)) B_0^2 / 2 \mu_0 r^2$  at  $t = \tau \ln(2 + R_m) \approx \tau \ln 2$  (note, here,  $e^{-\nu \tau} \sim 0.5$ ).  $R_m \ll 1$  (or  $\tau \gg \tau \nu$ ) is usually satisfied and the error is negligible around the maximum. In Equation (11) the ratio  $P_i/P_0$  (as defined in the explanation on Equation (1)) is obtained to be  $R_m/2$ , which indicates the term  $P_i$  can be neglected safely. Then Equation (11) is changed to the more simplified form Equation (12).

$$P = - \frac{\alpha^2 B_0^2 R_p^2 \sigma t_v}{r^2 \tau (1 - \tau \nu)} \frac{F}{G} (1 - e^{-\nu \tau}) e^{-\nu \tau} \quad (12)$$

$$\approx - \frac{\alpha^2 B_0^2 R_p^2 \sigma t_v}{r^2 \tau} \frac{F}{G} (1 - e^{-\nu \tau}) e^{-\nu \tau} \quad (12)'$$

The maximum of Equation (12)',  $P_m$ , is easily solved and is given by Equation (13) which is used generally for fast estimation of the pressure..

$$P_m = - \frac{\alpha^2 B_0^2 R_p^2 \sigma t_v}{4 \mu_0 r^2 \tau} \quad (13)$$

Discharging toroidal field coil is equivalent to simple dump of the coil current to a resistor, and toroidal field decays exponentially as  $B_t = (B_0 R_p / r) e^{-\nu \tau}$ . The vessel current has the same form as that of charging stage except that the magnitude is less by  $1/\alpha$  and the sign is plus. The pressure is given by Equation (14), slightly different from Equation (8).

$$P = \frac{B_0^2 R_p^2 \sigma t_v}{r^2 \tau (1 - \tau \nu)} \frac{F}{G} \left( e^{-\nu \tau} + \frac{\mu_0 \sigma t_v}{2 \tau (1 - \tau \nu)} \frac{F}{G} \right) (e^{-\nu \tau} - e^{-\nu \tau \nu}) (e^{-\nu \tau} - e^{-\nu \tau \nu}) \quad (14)$$

And if again  $\tau \gg \tau \nu$ , we get following equation.

$$P = \frac{B_0^2 R_p^2 \sigma t_v}{r^2 \tau} \frac{F}{G} \left( 1 + \frac{\mu_0 \sigma t_v}{2 \tau} \frac{F}{G} \right) e^{-2\nu \tau} \quad (15)$$

The maximum pressure is  $R_p^2 R_m (2 + R_m) B_0^2 / 2 \mu_0 r^2$  at  $t=0$ . It is about  $4/\alpha^2$  times larger than that of charging phase.

F in Equation (5) and G in Equation (6) have a variable  $a(\theta)$  calculated along the boundary of the vessel cross-section. If the boundary is described with single- or multi-function well defined, F and G may be solved analytically, otherwise they are obtained numerically. Let's consider a practical example with the design parameters of the vessel of the KT-2 device. Figure 2 shows details of the vessel boundary which consists of three arcs and one straight[5]. For solving F it is convenient to convert the double integral to a single one as depicted in Fig. 3. Then, F is solved as Equation (16).

$$\begin{aligned}
 F = \int \frac{dA}{r} &= 2 \int_0^\epsilon \frac{a_1^2 \sin^2 \theta d\theta}{R_1 + a_1 \cos \theta} + 2 \int_\epsilon^\pi \frac{a_2 \sin \theta (a_2 \sin \theta + z_2) d\theta}{R_2 + a_2 \cos \theta} = \\
 &2(R_1 \theta - a_1 \sin \theta - 2\sqrt{(R_1^2 - a_1^2)} \tan^{-1} \frac{\tan(\theta/2)\sqrt{(R_1^2 - a_1^2)}}{R_1 + a_1}) \Big|_0^\epsilon + \\
 &2(R_2 \theta - a_2 \sin \theta - 2\sqrt{(R_2^2 - a_2^2)} \tan^{-1} \frac{\tan(\theta/2)\sqrt{(R_2^2 - a_2^2)}}{R_2 + a_2} - z_2 \ln(R_2 + a_2 \cos \theta)) \Big|_\epsilon^\pi \quad (16)
 \end{aligned}$$

G is given by Equation (17).

$$\begin{aligned}
 G = \int \frac{d\ell}{r} &= 2 \int_0^\epsilon \frac{a_1 d\theta}{R_1 + a_1 \cos \theta} + 2 \int_\epsilon^\pi \frac{a_2 d\theta}{R_2 + a_2 \cos \theta} + 2z_2/(R_2 - a_2) = \\
 &\frac{2a_1}{\sqrt{(R_1^2 - a_1^2)}} \tan^{-1} \frac{\tan(\theta/2)\sqrt{(R_1^2 - a_1^2)}}{R_1 + a_1} \Big|_0^\epsilon + \\
 &\frac{2a_2}{\sqrt{(R_2^2 - a_2^2)}} \tan^{-1} \frac{\tan(\theta/2)\sqrt{(R_2^2 - a_2^2)}}{R_2 + a_2} \Big|_\epsilon^\pi + 2z_2/(R_2 - a_2) \quad (17)
 \end{aligned}$$

Table 1 summaries some parameters of the vessel geometry, material and the toroidal field(refer to the notations in the Fig. 3 for geometry parameters)[2,3]. In this special case  $F=0.667$  and  $G=1.7657$ . The inductance and resistance of the vessel in the poloidal direction are  $1.334 \times 10^{-7}$  H and  $1.639 \times 10^{-5} \Omega$ . The time constant of the vessel as a electrical circuit,  $\tau_v$ , is 8.14 msec, which is much shorter than that of the toroidal coil,  $\tau$ , and  $R_m$  is about

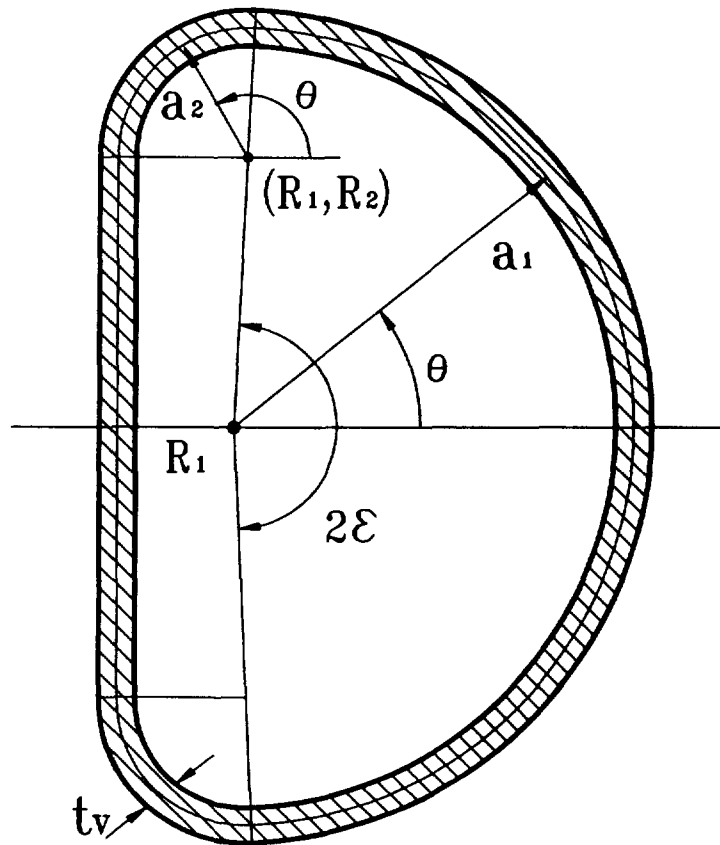


Fig.2. Schematic of the KT-2 vacuum vessel.

Table 1. Parameters for calculating induced vessel currents and pressures.

$\alpha$	1.5
$B_{t0}$	3 Tesla
$R_p$	1.4 m
$\tau$	2.86 sec
$t_v$	0.012 m
$R_1$	1.2495 m
$a_1$	0.6645 m
$R_2$	1.27 m
$a_2$	0.214 m
$z_2$	0.45 m
$\varepsilon$	87.4 ° (1.5254 radian)
$\eta$ (ss 304L)	$7 \times 10^{-7} \Omega \cdot m$
$R_v$	$1.639 \times 10^{-5} \Omega$
$L_v$	$1.334 \times 10^{-7} H$
$\tau_v$	8.14 msec



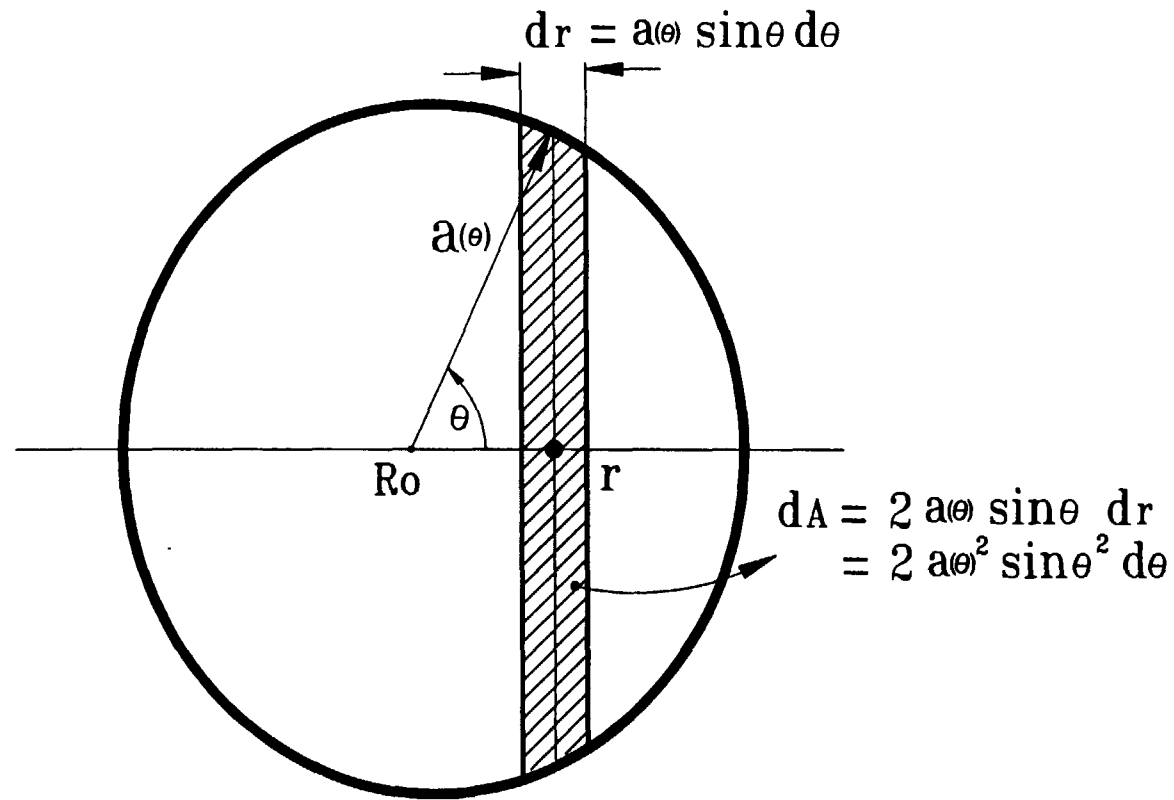


Fig.3. Model for surface integration over the vessel cross-section.

0.002845.

Evolution of the vessel eddy currents during toroidal coil charge for three values of the wall thickness is shown in Fig. 4. The currents increase abruptly from zero to the maximums in short times, order of  $\tau_v$ 's, and the profiles are very similar to simple exponential decays after the maximums.

The pressure acting on the vessel during field rise is produced by interaction of a basically decreasing current with a increasing field, therefore, there must be a maximum in the pressure profile as described in Fig. 5. The peak pressure depends on the wall thickness linearly in approximation, while the peak time is nearly constant for different thicknesses, determined by only the time constant of the coil, and is found around  $2(\approx 2.86\ln 2)$  sec, when each of the current and the field has a half of it's maximum. For reference, relative errors of Equations (11), (11)', (12) and (12)' compared with the exact solution, Equation (8), are shown in Fig. 6.

During coil discharge both the current(as described in Fig. 4 considering only the trend) and the field decrease at  $t > 0^+$  ( $\sim$  a few  $\tau_v$ ), then the maximum pressure appears at  $t = 0^+$  (as indicated in Fig. 7) and is given by combination of the maximum current and the maximum field. The peak value is, as mentioned above, nearly  $4/\alpha^2$  times higher than in the coil charge for the same wall thickness.

The pressure is inversely proportional to  $r^2$ , which may produces great unbalance in the spatial distribution of the pressure. The ratio,  $\kappa$ , of a maximum to a minimum in the pressure distribution is expressed as  $\kappa = ((A-1)/(A+1))^2$ , here A is an effective aspect ratio. Because KT-2 is a large-aspect-ratio device, the unbalance factor  $\kappa$  is not so large and is about 3.3 as illustrated in Figures 8 and 9 showing the spatial distributions of the maximum pressure acting on the vessel during coil charge and discharge, respectively. The pressures are expressed as  $-22500/r^2$  (inward) and  $+38690/r^2$  (outward) during the charging and discharging stages, respectively, for 12 mm thick wall.

### III. Diamagnetism change in a plasma disruption[6,7,8]

As the plasma loses it's energy due to some instabilities, the plasma temperature and the poloidal beta ( $\beta_p$ ) decrease steeply in a very short time of a few hundreds  $\mu$  sec followed by the main disruption. Although the real loss mechanism of the plasma energy is not well known, it is reasonable to consider the energy to be lost linearly during the full period.

The pressure-balance equation in the plasma which has a large aspect ratio ( $R_p/a_p \gg 1$ ,  $a_p$

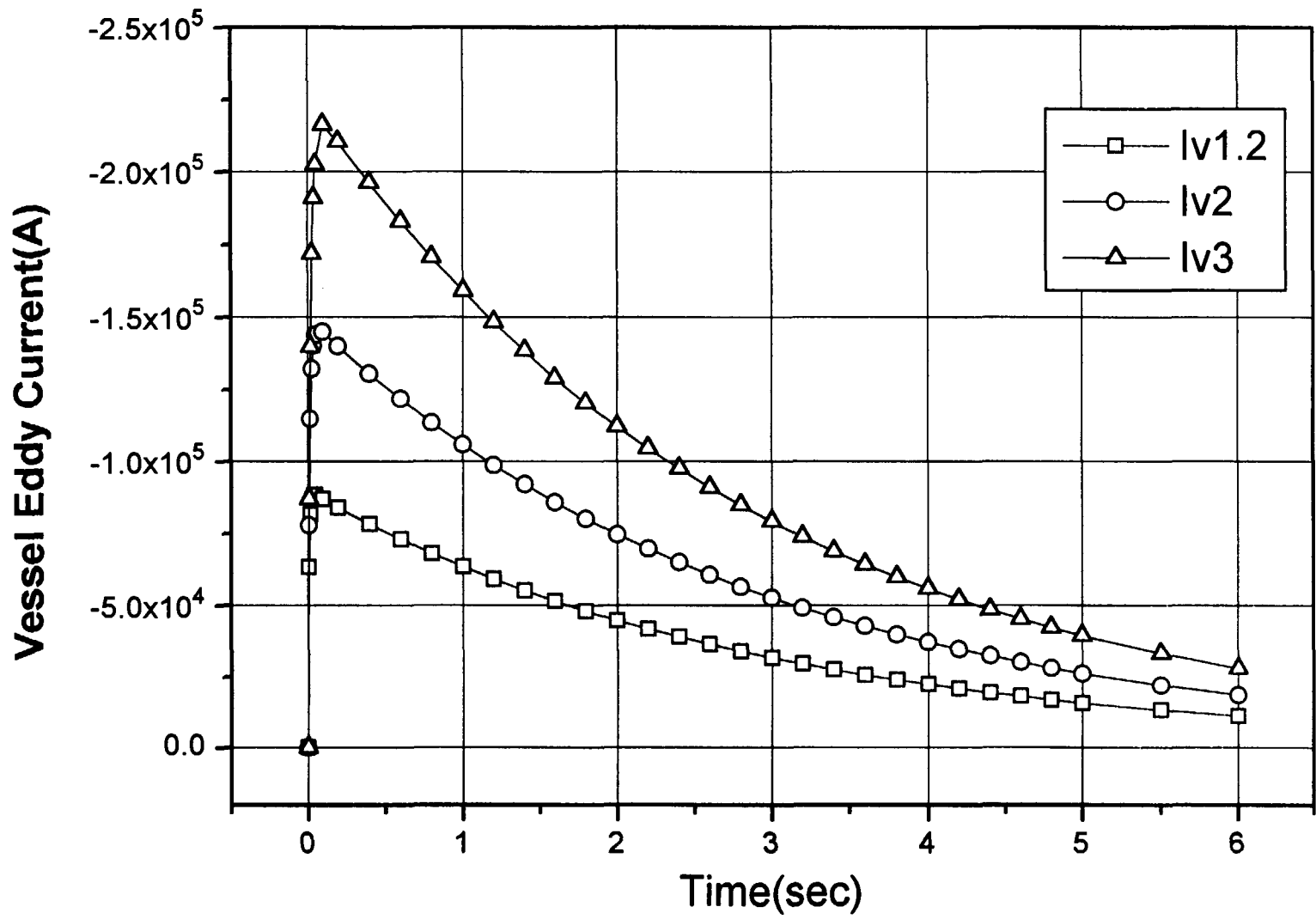


Fig. 4. Vessel currents induced during coil charge for three wall thicknesses(1.2,2 and 3 cm).

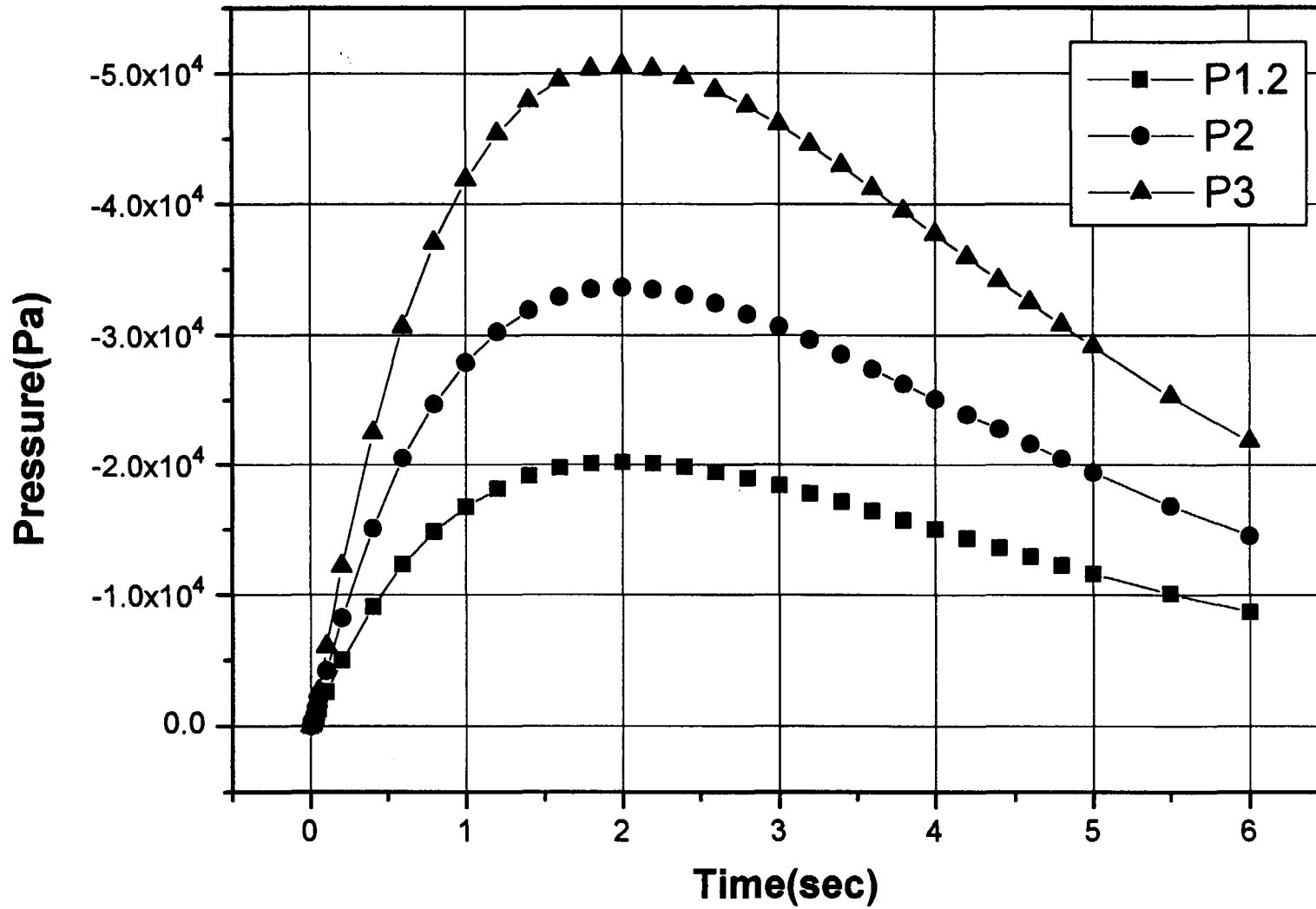


Fig. 5. Pressures acting on the vessel during coil charge for three wall thicknesses.

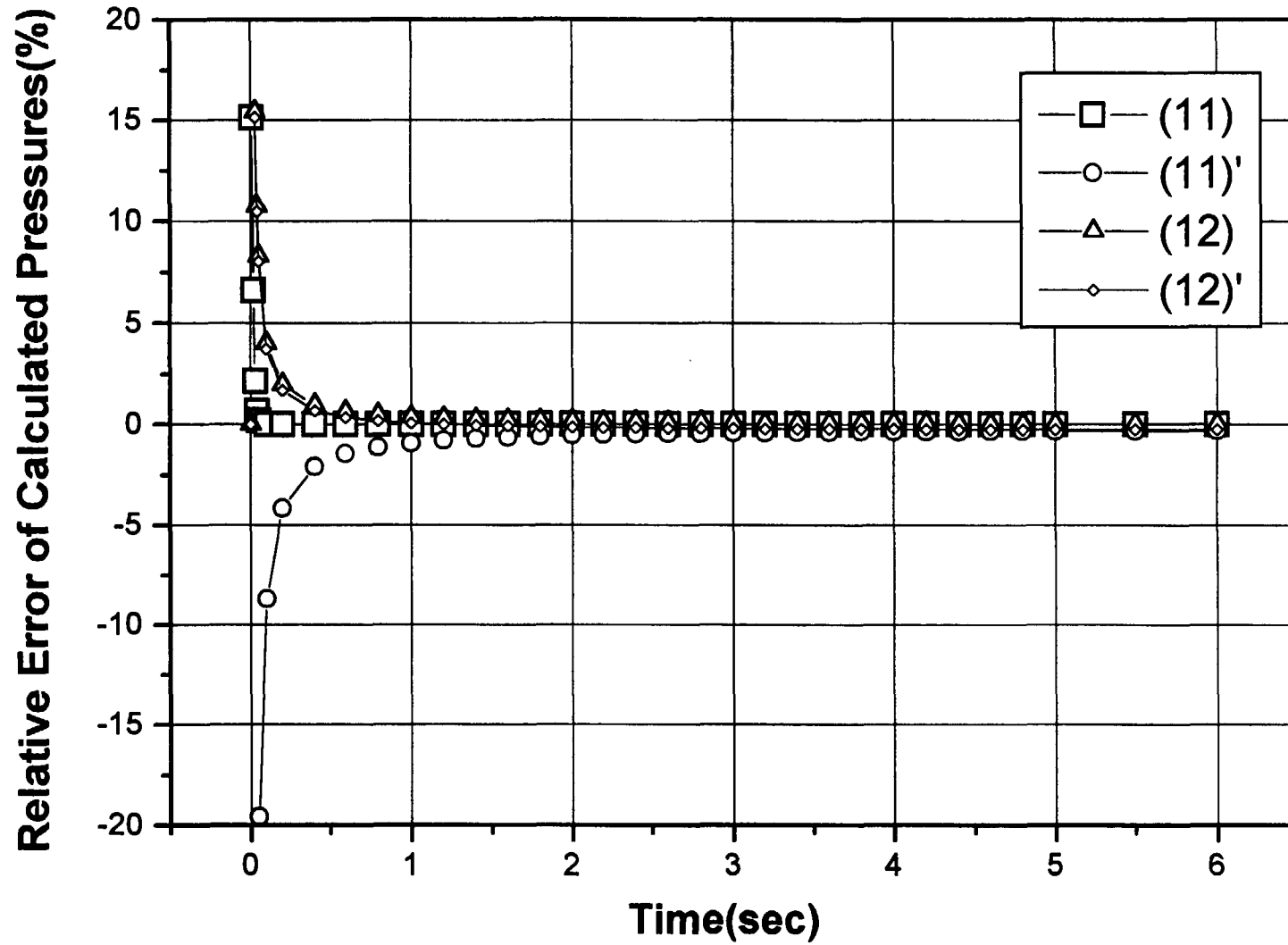


Fig. 6. Relative error of pressures calculated using approximate expressions(Eqs.11,11',12 and 12')

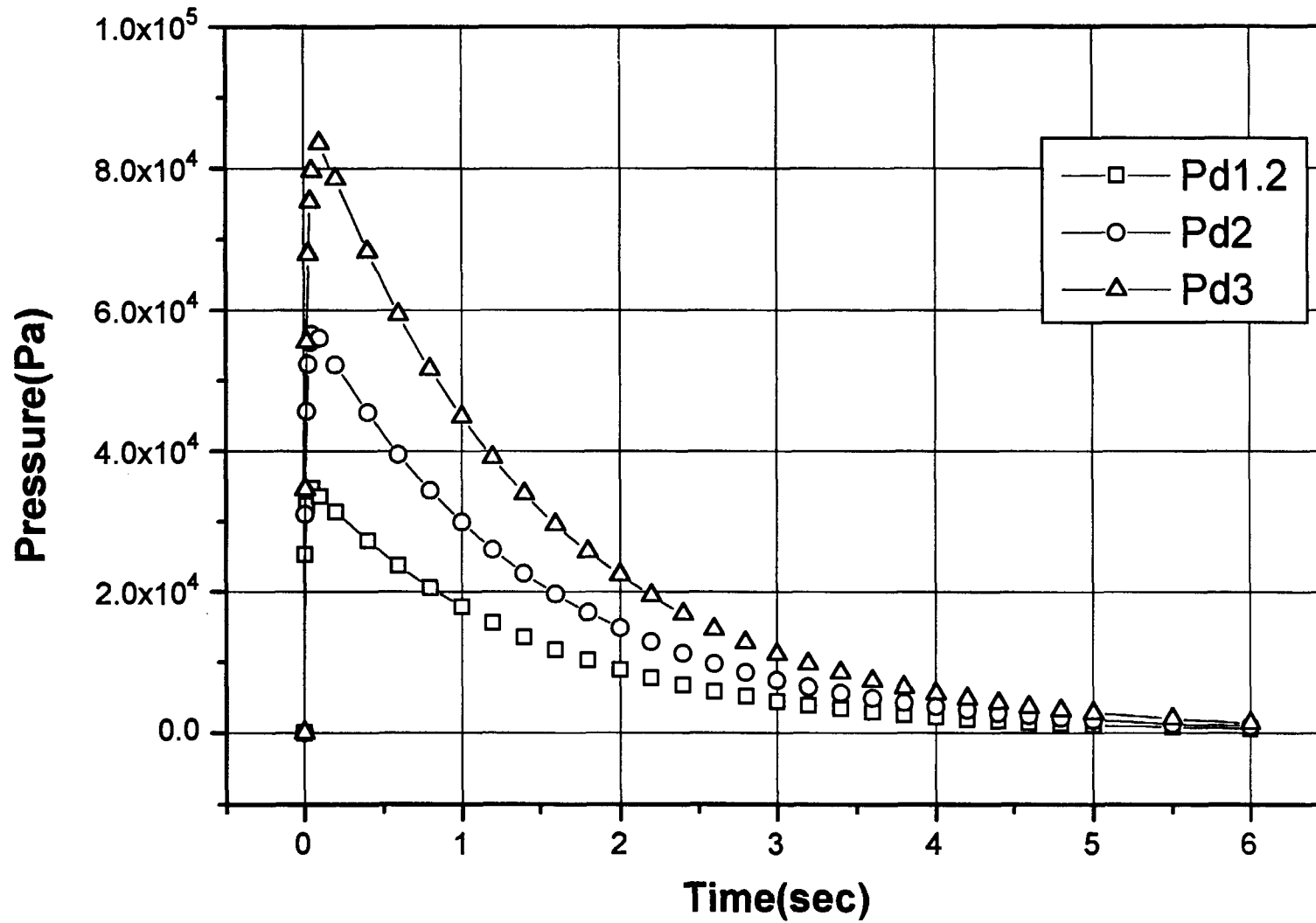


Fig. 7. Pressures acting on the vessel during coil discharge for three wall thicknesses.

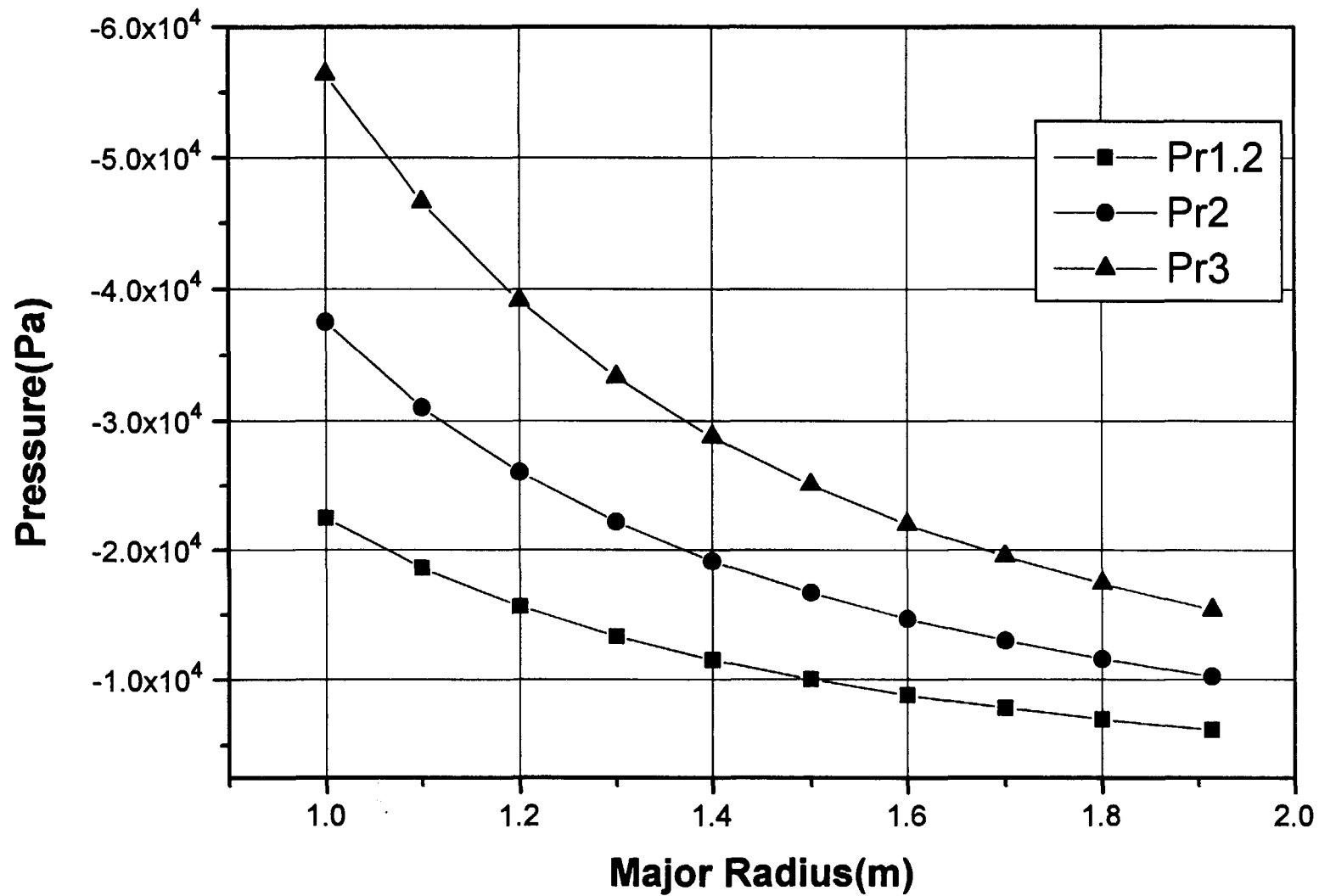


Fig. 8. Spatial variation of the maximum pressures acting on the vessel during coil charge.

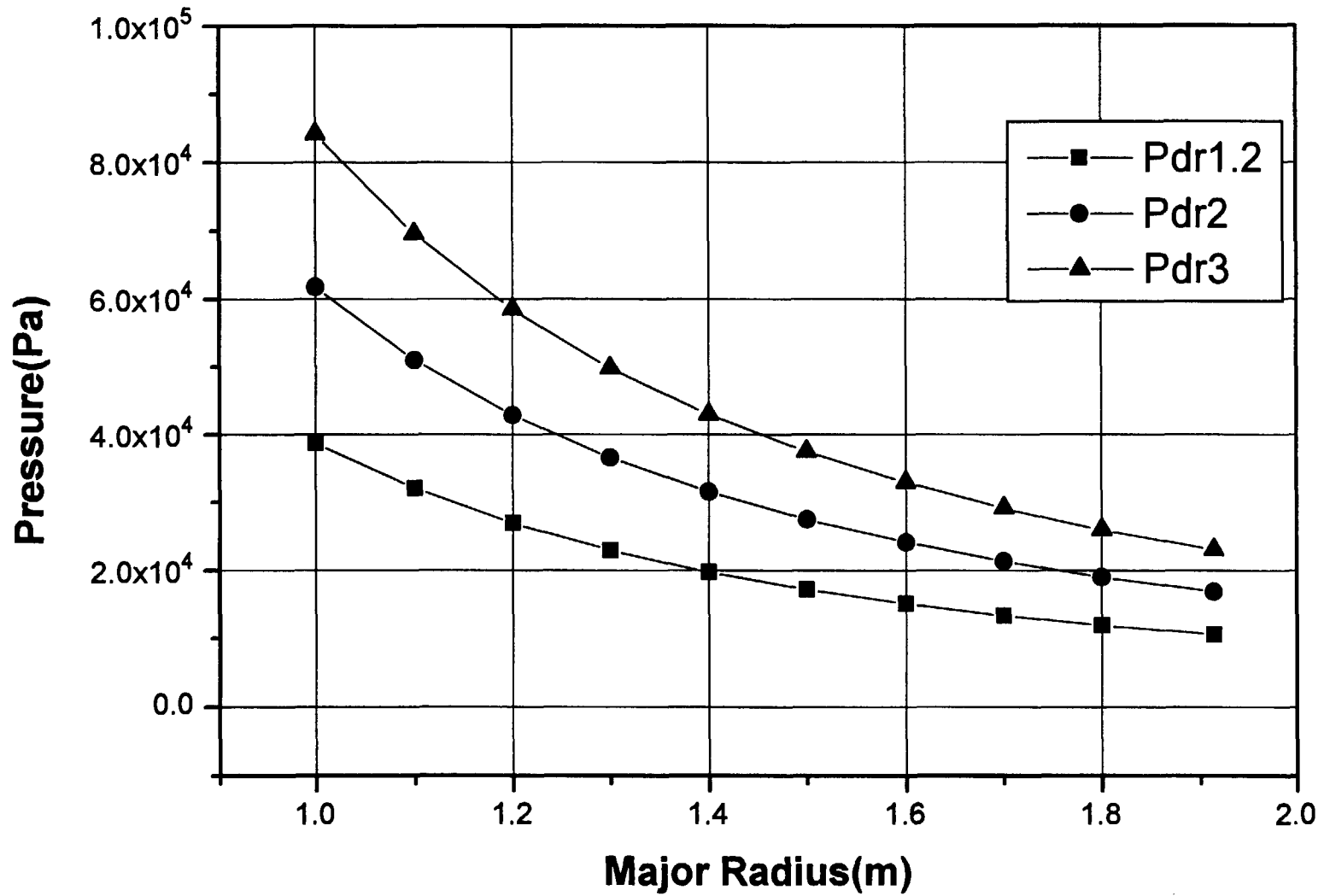


Fig. 9. Spatial variation of the maximum pressures acting on the vessel during coil discharge.



is the plasma minor radius) is written approximately in the following form.

$$\langle p \rangle \cong \frac{\langle\langle B_p \rangle\rangle_\psi^2}{2\mu_0} - \frac{\langle B_t^2 - B_{ta}^2 \rangle}{2\mu_0} \quad (18)$$

where,  $p$ : plasma pressure ( $= \sum n_k T_k$ ,  $n$  and  $T$  are density and temperature of  $k$ th species) [Pa]

$\langle \rangle$ : average over the volume containing the plasma, equivalent to the average over the plasma cross-section assuming axisymmetry.

$\langle\langle \rangle\rangle_\psi$ : average over the flux surface, equivalent to the average along the perimeter of the plasma cross-section assuming axisymmetry.

$\langle\langle B_p \rangle\rangle_\psi = \int \mathbf{B} \cdot d\mathbf{l} / \int dl = \mu_0 I_p / \int dl$ ,  $I_p$  is plasma current,  $\int dl$  is a line integral along the plasma periphery ( $\approx 2\pi a_0((1+k^2)/2)^{1/2}$ ),  $a_0$  and  $k$  are the short minor radius and elongation of the plasma cross-section.

$B_{ta}$ : toroidal field in air or vacuum.

$B_t$  inside the plasma is produced by internal poloidal current source as well as external toroidal field coils, while  $B_p$  is produced by the toroidal plasma current. For given plasma current and the plasma pressure profiles,  $\langle B_t^2 - B_{ta}^2 \rangle$  is adjusted in such a way that the pressure-balance equation is satisfied. If we define  $\Delta\Phi$  as flux change due to plasma diamagnetism or paramagnetism, then  $\Delta\Phi = \int (B_t - B_{ta}) ds \equiv \langle\langle B_t - B_{ta} \rangle\rangle_s$  where  $s$  denotes the plasma cross-section. As  $\langle B_t^2 - B_{ta}^2 \rangle = \langle (B_t + B_{ta})(B_t - B_{ta}) \rangle \approx 2\langle B_{ta}(B_t - B_{ta}) \rangle = 2\pi R_p B_{t0} \langle\langle B_t - B_{ta} \rangle\rangle_s / V \approx 2B_{t0} \Delta\Phi / S$ , where  $V$  is the volume of the plasma and  $S$  is the area of the plasma cross-section ( $S \approx \pi a_0^2 k(2J_1(\delta)/\delta)$ ),  $J_1(x)$  is the first order Bessel function of the first kind and  $\delta$  is triangularity of the plasma cross-section,  $V = 2\pi R_p(1 - 0.233\delta/A)S \cong 2\pi R_p S$ , Equation (18) is arranged as Equation (19).

$$\Delta\Phi = \frac{\mu_0^2 I_p^2}{8\pi B_{t0}} \frac{(4kJ_1(\delta)/\delta)}{(1+k^2)} (1 - \beta_p) \quad (19)$$

where,  $\beta_p$ : average poloidal beta ( $= 2\mu_0 \langle p \rangle / \langle\langle B_p \rangle\rangle^2$ ; plasma kinetic energy  $\div$  magnetic energy)

$\Delta\Phi > 0$  if  $\beta_p < 1$  (paramagnetic; increase of toroidal magnetic field), and  $\Delta\Phi < 0$  if  $\beta_p > 1$  (diamagnetic; decrease of toroidal magnetic field) as illustrated in Fig. 10.

In the toroidal device there exists always a toroidal magnetic field. Without plasma there is no change of the toroidal magnetic flux ( $\Delta\Phi = 0$ ). As plasma is generated (possibly by RF)

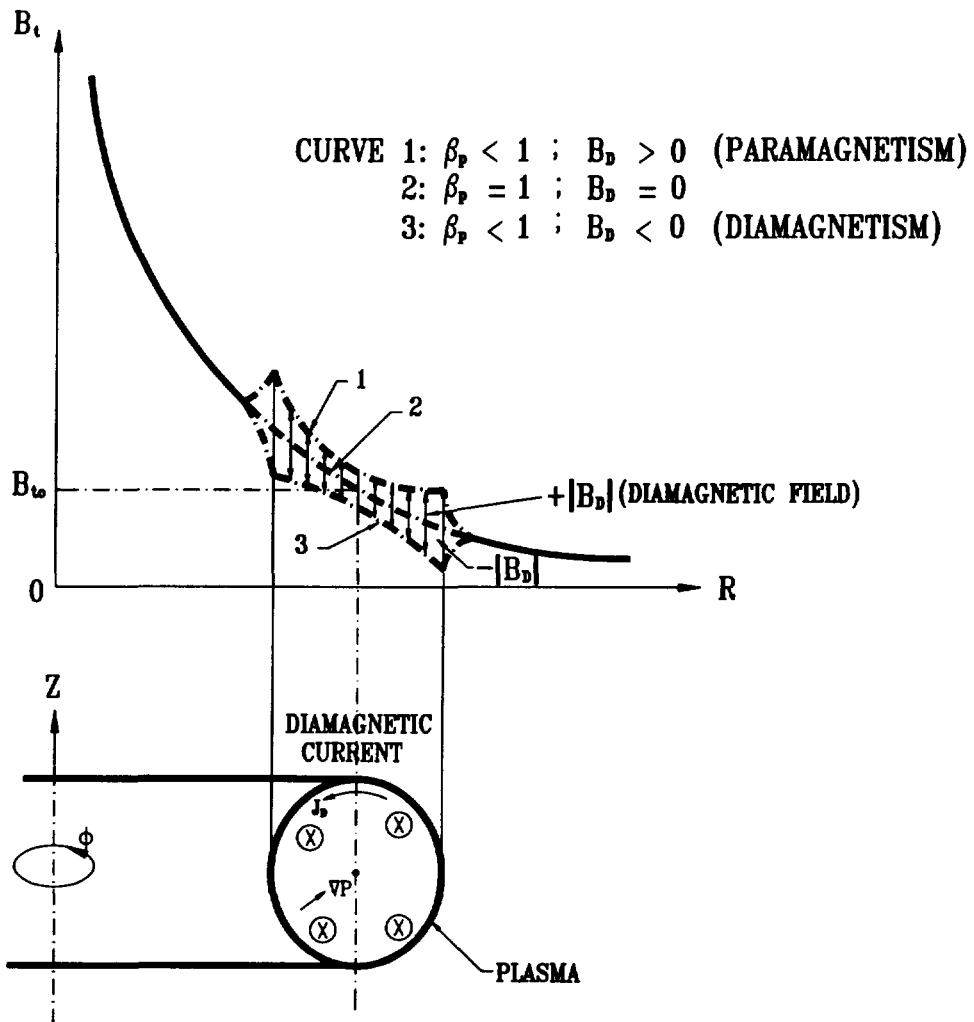


Fig.10. Diamagnetism change in the tokamak plasma.

in the magnetic field, diamagnetic current induced by the radial gradient of the plasma pressure starts to flow and produces additional toroidal fields antiparallel to the original one, which represents there may be abrupt change of the toroidal flux. If the plasma is initiated by a toroidal loop voltage there must be a toroidal current producing poloidal magnetic fields whose radial gradient can make paramagnetic poloidal current. In a cold plasma ( $\beta_p \approx 0$ ), as just generated before sufficient heating, paramagnetic current is larger than diamagnetic one, and  $\Delta \Phi > 0$ , but the magnitude is expected to be small because of low plasma current.

As the plasma (toroidal) current develops further, flux change depends only on the current, that is  $\Delta \Phi \sim I_p^2$ . If the plasma is heated up sufficiently by RF or NBI the diamagnetic current increases and takes over the paramagnetic one, the net change of  $\Delta \Phi$  is negative ( $\beta_p > 1 \rightarrow \Delta \Phi < 0$ ).

In the plasma disruption the current quench, relatively slow phenomenon, follows the energy quench[9]. During the energy quench the plasma current is assumed to be not changed, but the poloidal beta decreases to zero, which causes increase of  $\Delta \Phi$ , while during the current quench the plasma current decays to zero, which results in decrease of  $\Delta \Phi$ . Current ramp down in the end of a normal mode may cause an increase of the poloidal beta, which make the flux change complicate, but it is not easy to treat it theoretically here. Figure 11 shows typical relation of the variations of the plasma current, diamagnetic signal and poloidal beta[7].

When the plasma poloidal beta changes from  $\beta_{pi}$  to  $\beta_{pf}$  in  $\Delta t_e$ , the vessel current induced during a plasma disruption,  $I_v$ , is given by following equation.

$$I_v = - \frac{\delta \Phi}{R_v \Delta t_e} (1 - e^{-\Delta t_e / \tau_v}) = - \frac{\delta \Phi \tau_v}{L_v \Delta t_e} (1 - e^{-\Delta t_e / \tau_v}) \quad (20)$$

where,  $\delta \Phi = \Delta \Phi_f - \Delta \Phi_i$ , suffix i and f denote initial and final values,  $\Delta \Phi_f = xx(1 - \beta_{pf})$ ,  $\Delta \Phi_i = xx(1 - \beta_{pi})$ , finally  $\delta \Phi = xx(\beta_{pi} - \beta_{pf})$  (refer to Equation (19) for the exact expression of the term xx).

If  $\Delta \Phi$  is non-zero during  $\Delta t_e$  and assuming  $\Delta t_e \ll \tau_v$ , the maximum vessel current  $I_{vm}$  is approximately given by  $\delta \Phi / L_v$ . This means that flux change is almost converted to magnetic energy of the vessel rather than resistive consumption during  $\Delta t_e$ . And note that  $I_v$  has minus sign if  $\beta_{pi} > \beta_{pf}$ , and vice versa. Then using Equations (1), (19) and (20) the pressure is expressed as Equation (21).

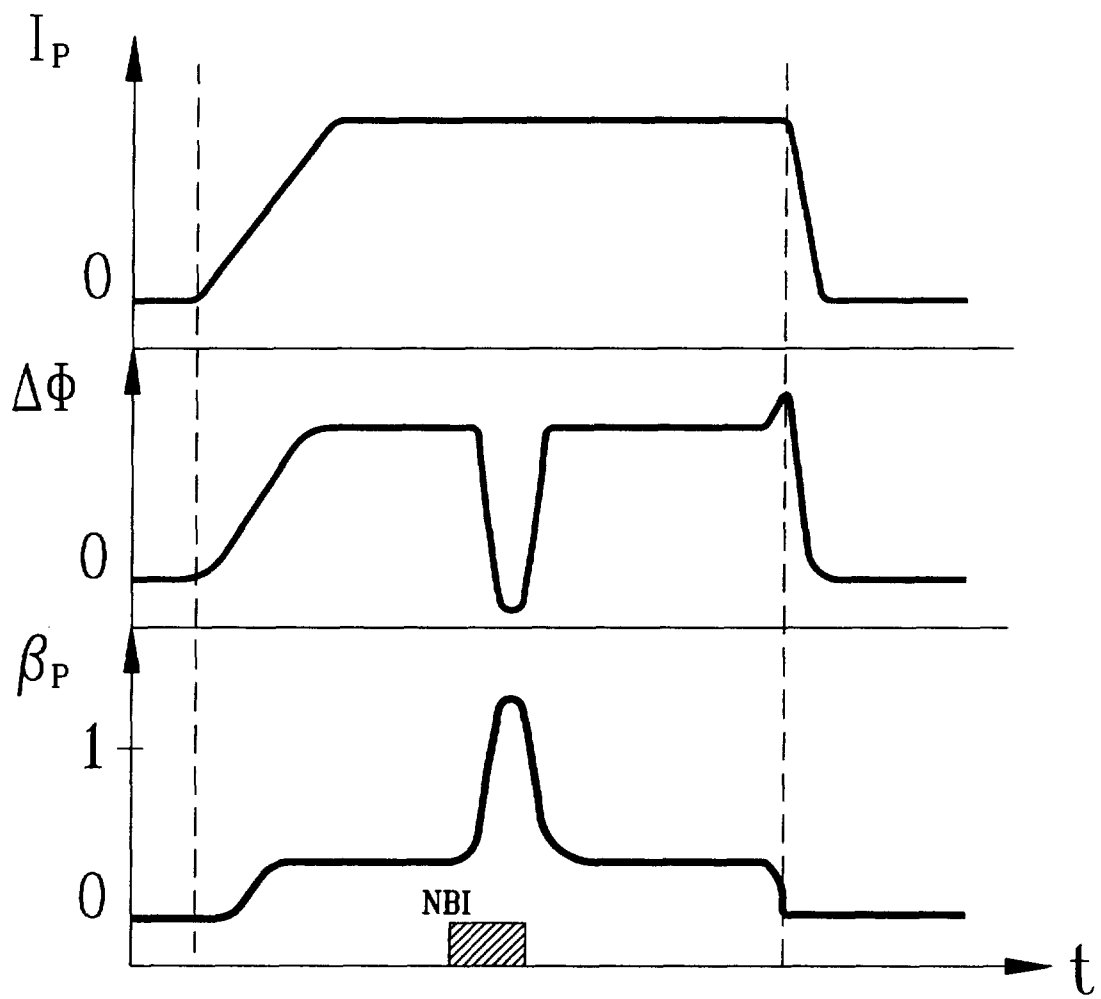


Fig.11. Variations of the plasma current, diamagnetism and poloidal beta.

$$\begin{aligned}
P &= - \frac{\delta \Phi}{2 \pi r L_v} \left( \frac{B_{t0} R_p}{r} + \frac{\mu_0 \delta \Phi}{4 \pi r L_v} \right) \\
&= - \frac{\mu_0^2 I_p^2 (\beta_{pi} - \beta_{pf}) \lambda R_p}{16 \pi^2 L_v r^2} + \frac{\mu_0^3 I_p^4 (\beta_{pi} - \beta_{pf})^2 \lambda^2}{256 \pi^4 B_{t0}^2 L_v^2 r^2}
\end{aligned} \tag{21}$$

where,  $\lambda = 4k J_1(\delta) / \delta (1+k^2)$

The second term of RHS of Equation (21) is much less than the first term, it is, therefore, neglected in the sample calculation using the parameters of KT-2 which is summarized in Table 2.

$\delta \Phi$  is calculated during the plasma disruption for  $\beta_{pi}=2.3$  to be  $9.77 \times 10^{-3}$  Wb, assuming  $\beta_{pf}$  is reasonably 0, and  $I_{vm}$  is  $-7.325 \times 10^4$  A. Then the pressure acting on the vacuum vessel is expressed as  $-48965/r^2$ , which implies the inward pressure.

If  $\beta_p$  is nearly zero or constant and there is only a linear change of the plasma current  $I_p$  as  $I_p(t) = I_i + (I_f - I_i)t / \Delta t_k = I_i + \gamma t$  ( $k=r$  for current rising,  $k=c$  for current quench,  $I_i$  is the initial current,  $\gamma$  is the change rate of the plasma current),  $I_v$  and  $P$  are given as below.

$$I_v = - \frac{2Q}{R_v} \gamma \left( (I_i - \gamma \tau_v)(1 - e^{-t/\tau_v}) + \gamma t \right) \tag{22}$$

$$P = - \frac{2Q}{R_v} \frac{B_{t0} R_p}{2 \pi r^2} \gamma \left( (I_i - \gamma \tau_v)(1 - e^{-t/\tau_v}) + \gamma t \right) \tag{23}$$

where,  $Q = \mu_0^2 \lambda (1 - \beta_p) / 8 \pi B_{t0}$ .

If  $\gamma > 0$  (during plasma initiation), it is easily found that  $I_v$  increases monotonically, then we can obtain approximate forms of Equation (22) as follows:

for  $t \ll \tau_v$

$$I_v \approx - \frac{2Q \gamma (I_i + \gamma t/2)}{L_v} t \tag{24}$$

and for  $t \gg \tau_v$

$$I_v \approx - \frac{2Q \gamma^2}{R_v} t = - \frac{2Q \gamma}{R_v} I_p \tag{25}$$

If  $\gamma < 0$  (during plasma disruption), there is a maximum in  $I_v$  and  $P$  at  $t = \tau_v \ln(1 - I_i / \gamma \tau_v)$ , as given by Equations (26) and (27).

$$I_v = - \frac{2Q\gamma}{R_v} (I_i + \gamma \tau_v \ln(1 - I_i / \gamma \tau_v)) \quad (26)$$

$$P = - \frac{2Q\gamma}{R_v} \frac{B_0 R_p}{2\pi r^2} (I_i + \gamma \tau_v \ln(1 - I_i / \gamma \tau_v)) \quad (27)$$

For two extreme cases 1)  $I_i / \gamma \ll \tau_v$  and 2)  $I_i / \gamma \gg \tau_v$ , Equation (26) may be approximated to following equations.

$$1) \quad I_v \approx \frac{Q I_i^2}{L_v} \quad (28)$$

$$2) \quad I_v \approx - \frac{2Q\gamma}{R_v} I_i \quad (29)$$

Equation (28) can be also derived by the relation  $\Delta I_v = \Delta \Phi / L_v = Q I_i^2 / L_v$  (when  $I_p$  changes from  $I_i$  to 0). Equation (29) is basically equivalent to Equation (25).

Using the parameters of Table 2 and assuming  $\gamma = 1$  MA/s in the start-up,  $\gamma = -500$  MA/s in the disruption, and  $\beta_p = 0$  in the both stages, variations of the vessel current are described as Fig. (12) and Fig. (13), respectively. Maximum induced vessel currents are calculated to be -1019 A during current rise in Equation (22), and 29448 A during current quench in Equation (26). The pressure exerting on the vessel due to diamagnetism change by current rise and current quench during the plasma initiation and disruption are expressed as  $-682/r^2$  and  $19684/r^2$ , respectively. Table 3 summarizes whole results obtained above.

The non-uniform spatial distribution of the pressure and the toroidal geometry of the tokamak vessel leads to non-vanishing horizontal forces exerting on the vessel. The pressures obtained above all have the form of  $P_0/r^2$  ( $P_0 \approx I_v B_0 R_p / 2\pi$ ). Horizontal force (centering or expanding) is obtained by integration of the radial component of the normal pressure as illustrated by Equation (30).

$$F_r = \int 2\pi r P \cos \theta \, dl = \int 2\pi r P \cos \theta \, a \, d\theta = 2\pi \int P r \, dz = 2\pi P_0 \int dz/r \quad (30)$$

If any functional expression for the plasma boundary is given,  $F_r$  is calculated analytically or

Table 2. Plasma parameters related with the plasma beta change.

$I_p$	500 kA
$R_p$	1.4 m
$a_0$	0.5 m
$A$	5.6
$k$	1.8
$\delta$	0.6
$J_1(\delta)$	0.2867
$\lambda (=4kJ_1(\delta)/\delta(1+k^2))$	0.8114
$\beta_p$	2.3
$\beta_p/A$	0.45
$Q(= \mu_0^2 \lambda (1 - \beta_p)/8 \pi B_0)$	$1.7 \times 10^{-14} (\beta_p=0)$
$\Delta t_e(\text{energy quench})$	$\sim 0.5$ ms
$\Delta t_c(\text{current quench})$	$\sim 1$ ms
$\Delta t_r(\text{current rising})$	0.5 s

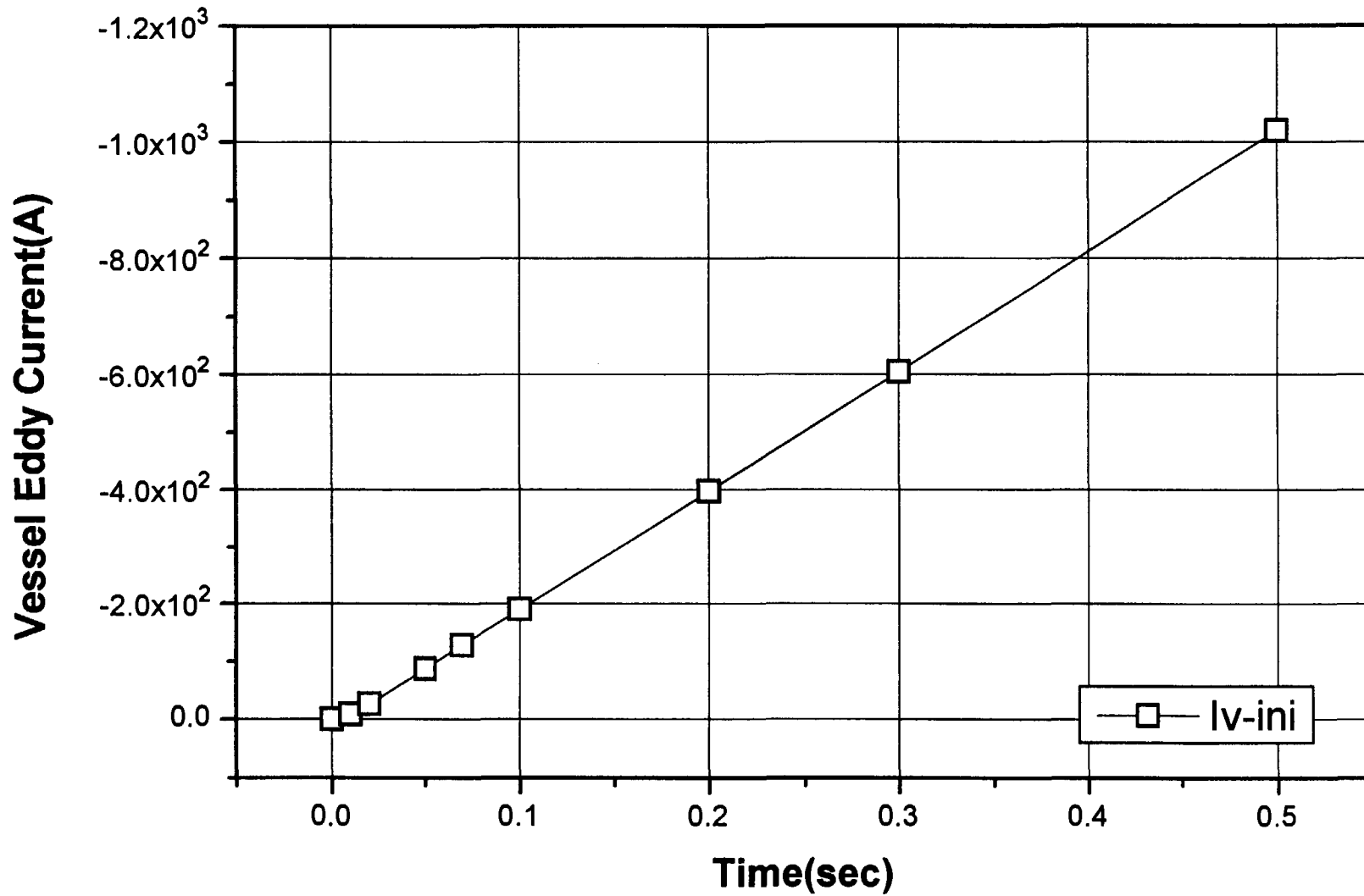


Fig.12. Variation of the vessel current induced by diamagnetism change during plasma initiation.



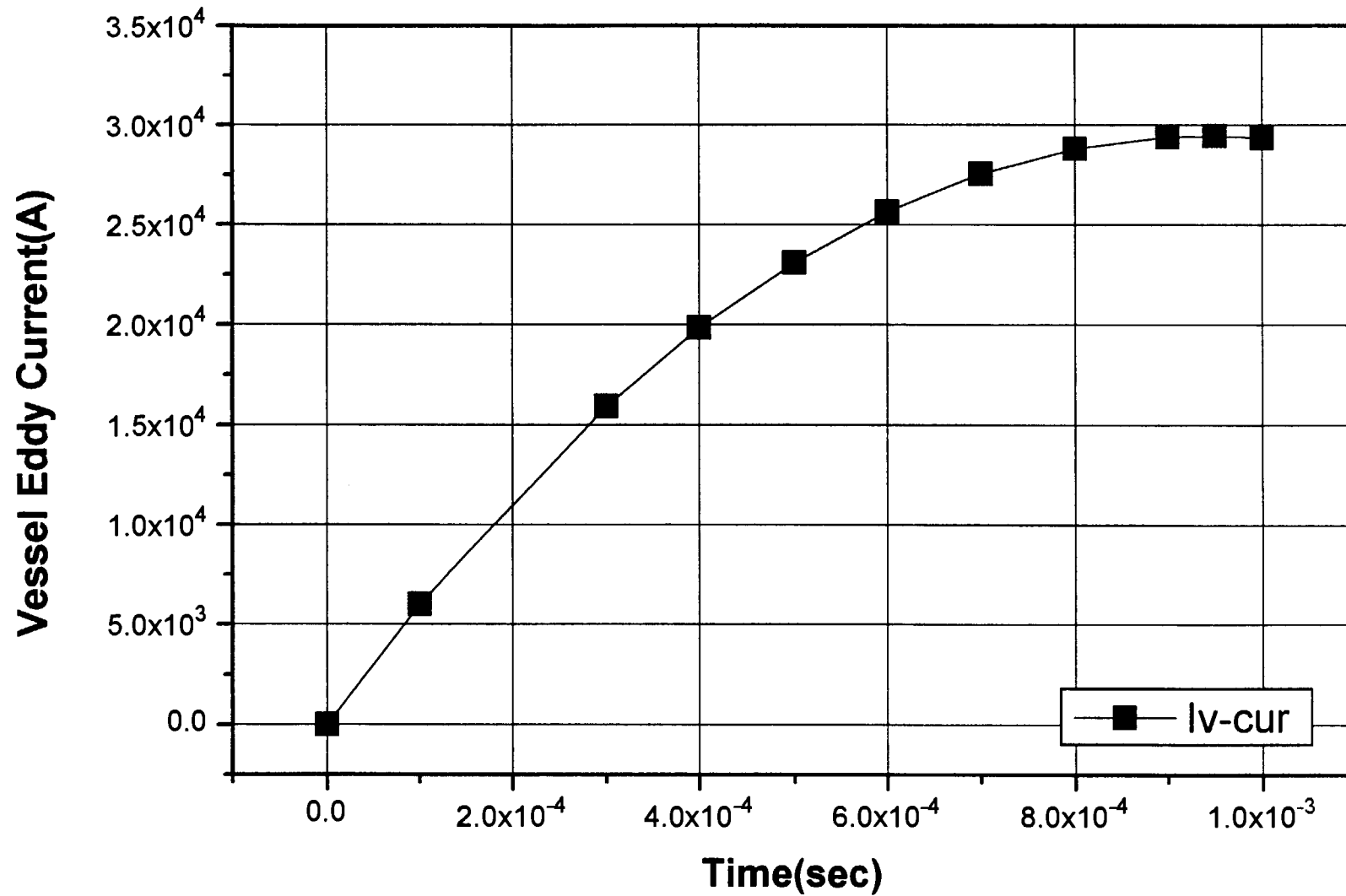


Fig.13. Variation of the vessel current induced by diamagnetism change due to current quench.

Table 3. The pressures induced on the vessel during toroidal field changes.

<b>TF change</b>	
-TF rising	$-22498/r^2$ Pa
-TF damping	$+38687/r^2$ Pa
<b>Diamagnetism</b>	
-energy quench during disruption	
$\beta_p = 2.3$	$-48965/r^2$ Pa
$\beta_p = 0$	$+21289/r^2$ Pa
-plasma current change( $\beta_p=0$ )	
· current rising $dI_p/dt = +1MA/s$	$-682/r^2$ Pa
· current decay in disruption $dI_p/dt = -500MA/s$	$+19684/r^2$ Pa

numerically. For the KT-2 vacuum vessel(refer to Fig. 2) we get following equation.

$$\begin{aligned}
F_r &= 4 \pi P_0 \left( \int_0^\epsilon \frac{a_1 \cos \theta d \theta}{R_1 + a_1 \cos \theta} + \int_\epsilon^\pi \frac{a_2 \cos \theta d \theta}{R_2 + a_2 \cos \theta} + \frac{z_2}{(R_2 - a_2)} \right) \\
&= 4 \pi P_0 \left( \left( \theta - \frac{2R_1}{\sqrt{R_1^2 - a_1^2}} \tan^{-1} \frac{((R_1 - a_1) \tan \theta / 2)}{\sqrt{R_1^2 - a_1^2}} \right) \Big|_0^\epsilon + \right. \\
&\quad \left. \left( \theta - \frac{2R_2}{\sqrt{R_2^2 - a_2^2}} \tan^{-1} \frac{((R_2 - a_2) \tan \theta / 2)}{\sqrt{R_2^2 - a_2^2}} \right) \Big|_\epsilon^\pi - z_2 / (R_2 - a_2) \right) \quad (31)
\end{aligned}$$

Using the parameters given at Table 1,  $F_r = -3.05P_0$ , which indicates that an inward pressure produces a net force directing the large major radius side, and vice versa (contrary to the trend for atmospheric pressure of uniform distribution). An approximate form of the radial force is obtained from Equation (30). That is,  $F_r = 2 \pi P_0 \int dz/r = 2 \pi P_0 \int rdz/r^2 \approx 2 \pi P_0 \int dA/R_0^2 = 2 \pi P_0 A_v / R_0^2$  ( $A_v$  is the cross-sectional area of the vessel).

We can also get an expression for vertical force  $F_v$  as follows;

$$\begin{aligned}
F_v &= 2 \pi P_0 \left( \int_0^\epsilon \frac{a_1 \sin \theta d \theta}{R_1 + a_1 \cos \theta} + \int_\epsilon^\pi \frac{a_2 \sin \theta d \theta}{R_2 + a_2 \cos \theta} \right) \\
&= -2 \pi P_0 \left( \ln(R_1 + a_1 \cos \theta) \Big|_0^\epsilon + \ln(R_2 + a_2 \cos \theta) \Big|_\epsilon^\pi \right) \quad (32)
\end{aligned}$$

As inserting KT-2 parameters to Equation (32), the relation  $F_v = +3.96P_0$  is obtained. Table 4 summaries the horizontal and vertical forces acting on the KT-2 vessel for the three modes producing the pressures of  $1/r^2$  form.

#### IV. Stress Analysis

The functional forms of the pressure acting on the vacuum vessel due to the poloidal eddy currents in any operation modes disussed above have the characteristic term of  $1/r^2$ . It is easily expected that similar patterns will be obtained in the stress distribution if it is caused by the forces of the same spatial variation. Therefore, stress analysis was performed for only

Table 4. The forces exerting on the vessel during toroidal field changes.

<b>TF rising</b>	$F_r$ +68691 Nt (centrifugal) $F_v$ -88998 Nt (compression)
<b>TF damping</b>	$F_r$ -118119 Nt (centripetal) $F_v$ +153038 Nt (tension)
<b>Diamagnetism</b>	
$\beta_p = 2.3$	$F_r$ +149342 Nt $F_v$ -193900 Nt
$\beta_p = 0$	$F_r$ -64931 Nt $F_v$ +84304 Nt
$di_p/dt=1MA/s$	$F_r$ +2080 Nt $F_v$ -2701 Nt
$di_p/dt=-500MA/s$	$F_r$ -60036 Nt $F_v$ +77949 Nt

two modes of TF damping and energy quench considering directions and magnitudes of the pressure(refer to Table 3).

The stress analysis was performed using 3D FEM code (ANSYS 5.2). ANSYS code consists of three processors: General Processor(/PREP7)-model generation, Solution Processor (/Solu)-define boundary conditions and solve the problem, General PostProcessor(/POST1)-estimate analysis results.

Table 5 summaries the parameters used in the stress analysis. Fig. 14 is the 3D finite element model describing a quarter of the KT-2 vacuum vessel. This model includes all ports installed in the 90° section and one support. All weldments are assumed to have the same material property with the original wall. It is also supposed that the vessel material is isotropic and elastic(Hooke's law is valid).

Figures 15 and 16 show 3D distributions of stress and deformation due to the atmospheric pressure applied uniformly on the vessel before tokamak discharge. The maximum stress appears at the boundary of the vessel wall and the NBI port to be  $0.612 \times 10^8$  Pa. It may be, however, much different from an average stress pattern, which is easily guessed from the fact that cool-colored pattern of relatively low stress covers almost whole surface of the model and hot-colored area of high stress is extremely localized as shown in Fig. 15. The maximum deformation is 0.241 mm.

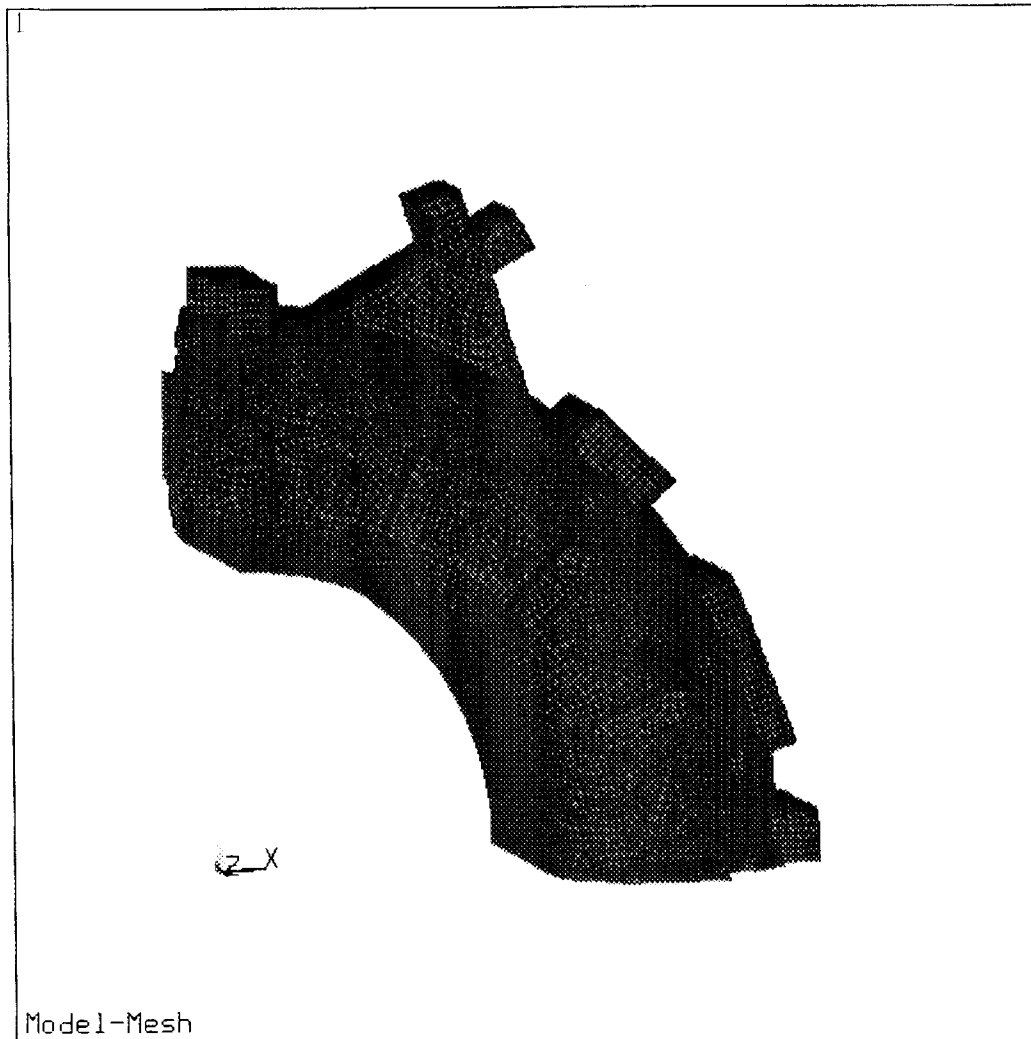
Figures 17 and 18 show the stress and deformation distributions on the vessel during TF damping. In this mode, the total mechanical load to the vessel is given as combination of the force due to TF damping itself and the atmospheric pressure. Because the induced pressure due to decrease of the toroidal field is outward which cancels much of the atmospheric pressure, the stress in this case is expected to moderate compared with the stress due to atmospheric pressure only. The majority of vacuum vessel has the stress in the range of  $0.620 \times 10^7 \sim 0.248 \times 10^8$  Pa. The maximum stress is  $0.558 \times 10^8$  Pa occurred at the welding line between the wall and the NBI port. This value is larger than 1/3 of the yield strength of s.s 304L, which means reinforcing structures must be added to the NBI port. The maximum deformation is 0.223 mm.

Fig. 19 is the stress distribution on the poloidal belt at a minor vessel cross-section far from the ports. This result is easily understandable and looks very reasonable comparing with 2D analysis(not shown here).

Figures 20 and 21 show 3D stress and deformation distributions due to diamagnetism change(increase of toroidal field) in the energy quench during a plasma disruption. Since the pressure caused by increase of the toroidal field is inward, then it adds to the atmospheric one. The stress on most part of vacuum vessel is in the range of  $0.756 \times 10^7 \sim 0.302 \times 10^8$  Pa.

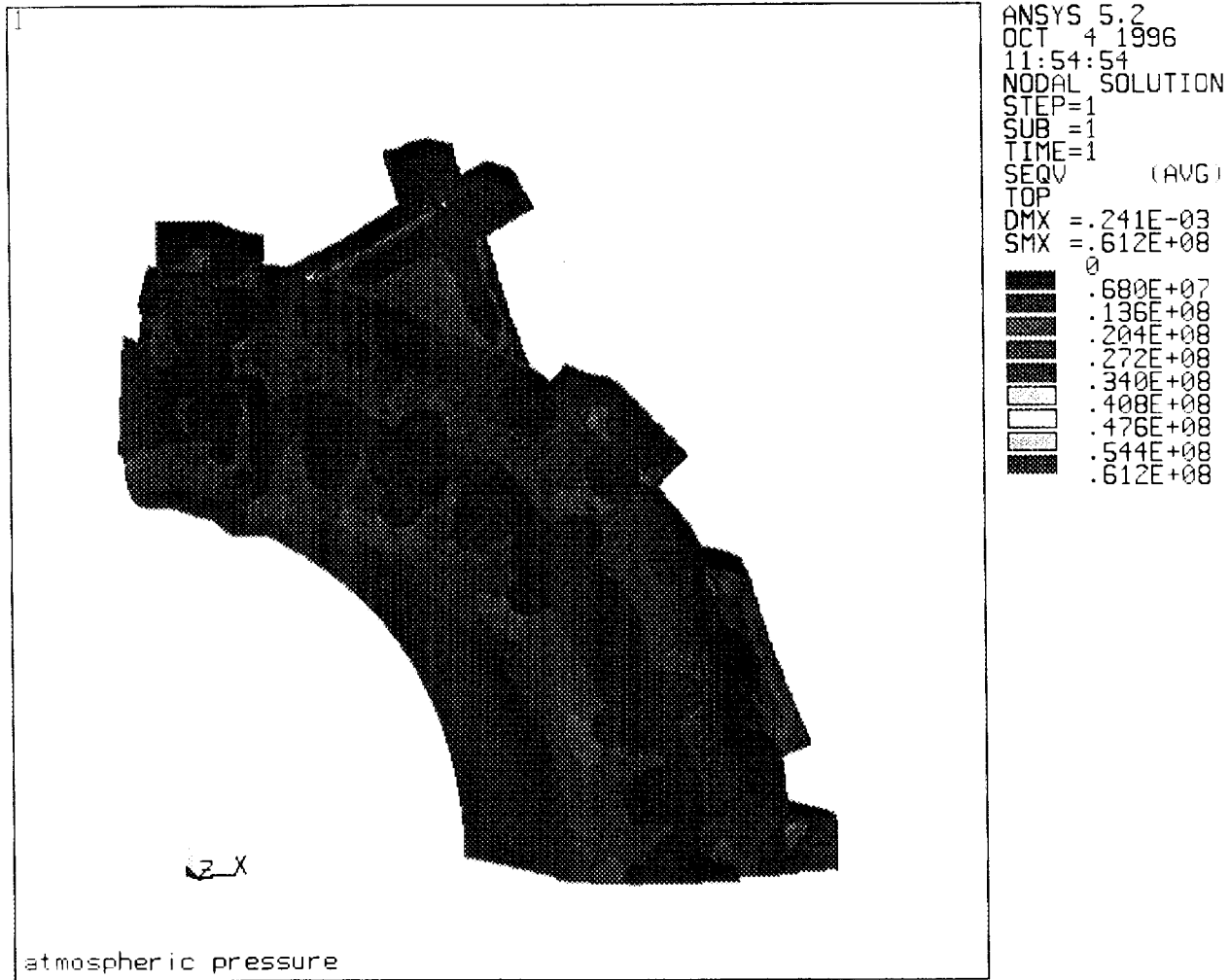
Table 5. Input parameters for the stress analysis.

Material	SUS304L , Isotropic
EMUNIT	MKS
ANTYPE	STATIC
Element Type	SHELL 63
Element Size	less than 3 cm
Element Numbers	15802 ea
Young's Modulus	$1.88 \times 10^{11}$ N/m <sup>2</sup>
Density	7900 kg <sub>m</sub> /m <sup>3</sup>
Poisson's ratio	0.27
Real constant	
. small circle port wall thickness	5 mm
. slim port wall thickness	10 mm
. vacuum vessel wall thickness	12 mm
. big, NBI port wall thickness	15 mm
. small,slim,NBI port flange thickness	25 mm
. big port flange thickness	30 mm
Main coordinate system	Cylindrical
Theta ; $\theta = 0^\circ , 90^\circ$	Symmetry ( quarter modeling )
Constraint	
. $64^\circ < \theta < 71^\circ , -94.2^\circ < \varphi < -58.5^\circ$	All DOF



ANSYS 5.2  
OCT 17 1996  
19:20:59  
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TYPE NUM  
  
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YV =.1202  
ZV =.9689  
\*DIST=1.725  
\*XF =1.087  
\*YF =1.128  
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A-ZS=.2253  
Z-BUFFER

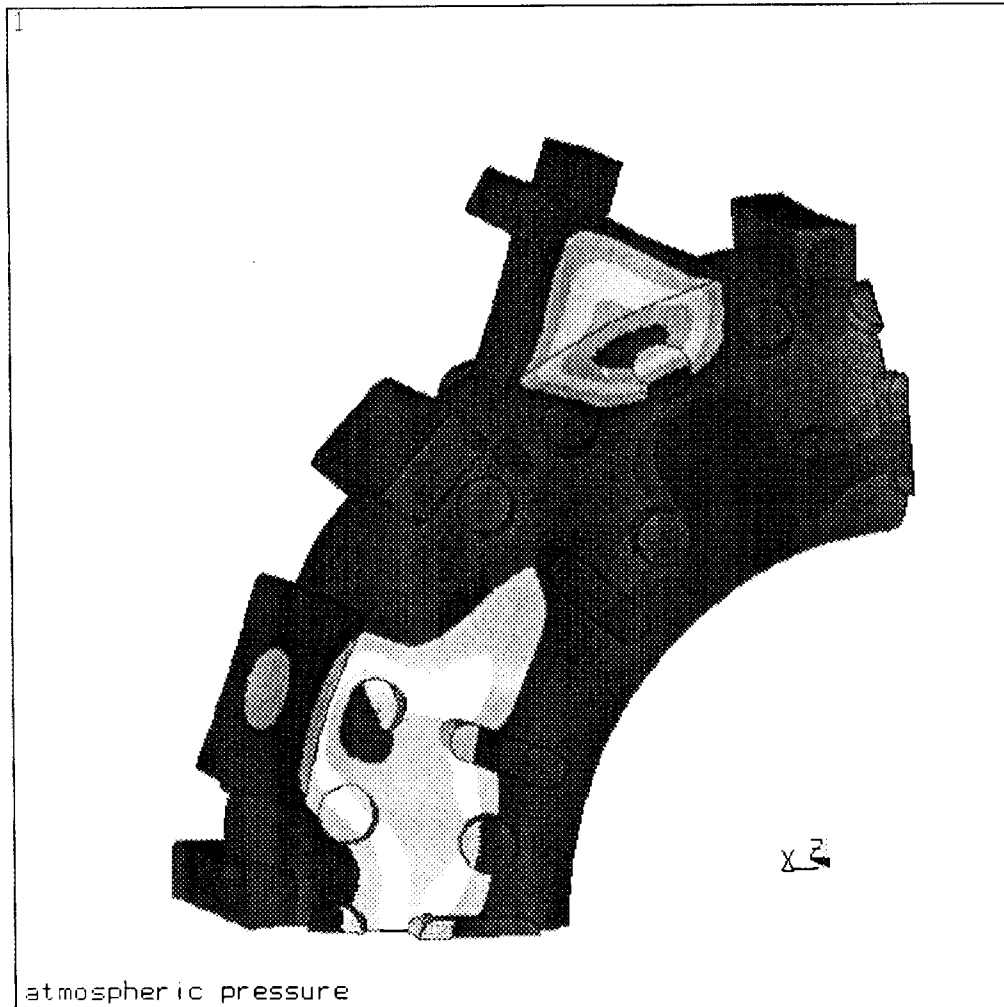
Fig. 14. 3D finite element Model of the KT-2 vacuum vessel.



34

Fig. 15. Stress distribution on the vessel by the atmospheric pressure.





```

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OCT  4 1996
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SUB  =1
TIME=1
USUM
TOP
RSYS=0
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SMX  =.241E-03
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.268E-04
.536E-04
.804E-04
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.188E-03
.214E-03
.241E-03

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Fig. 16. Deformation of the vessel by the atmospheric pressure.

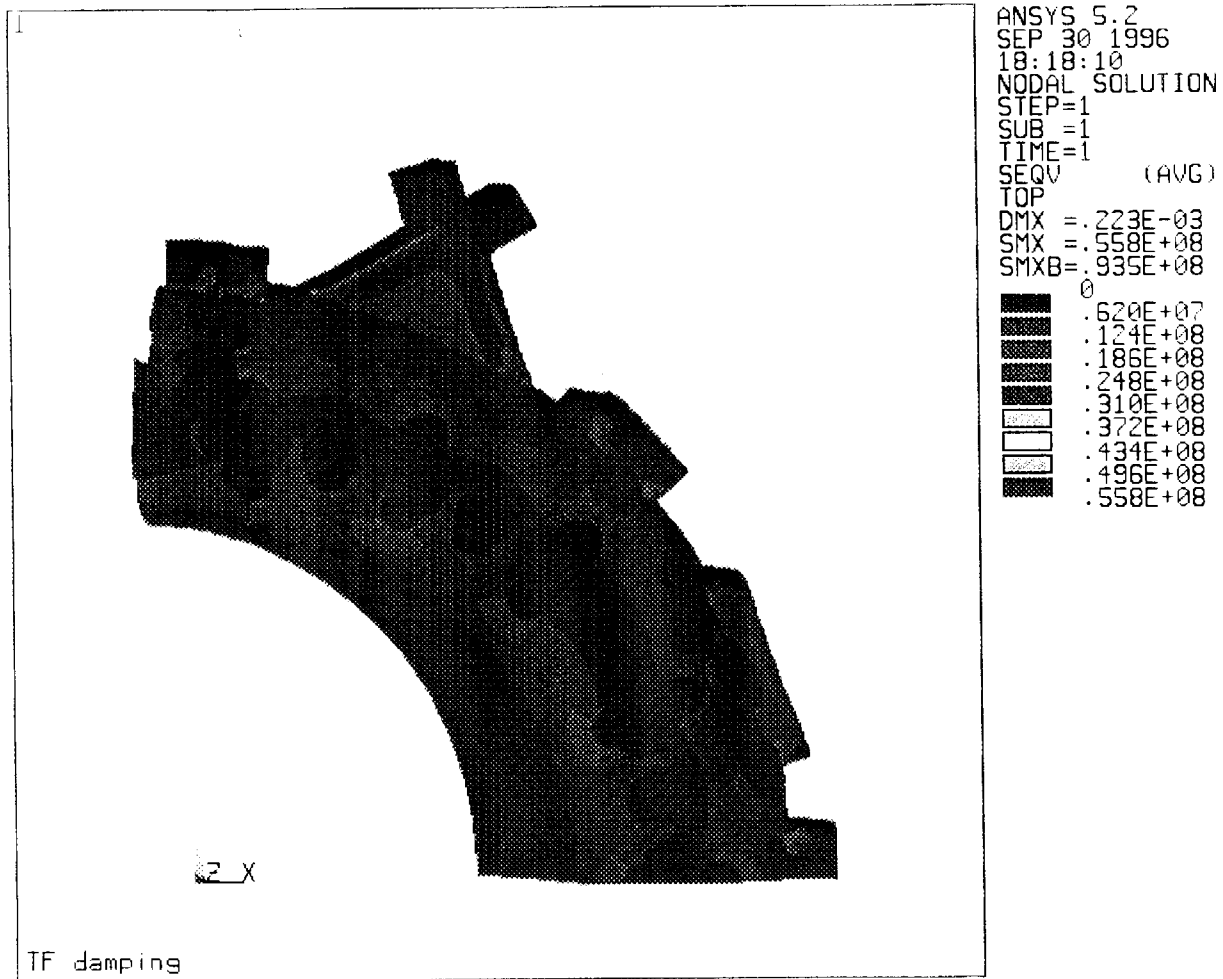


Fig. 17. Stress distribution on the vessel during TF damping.

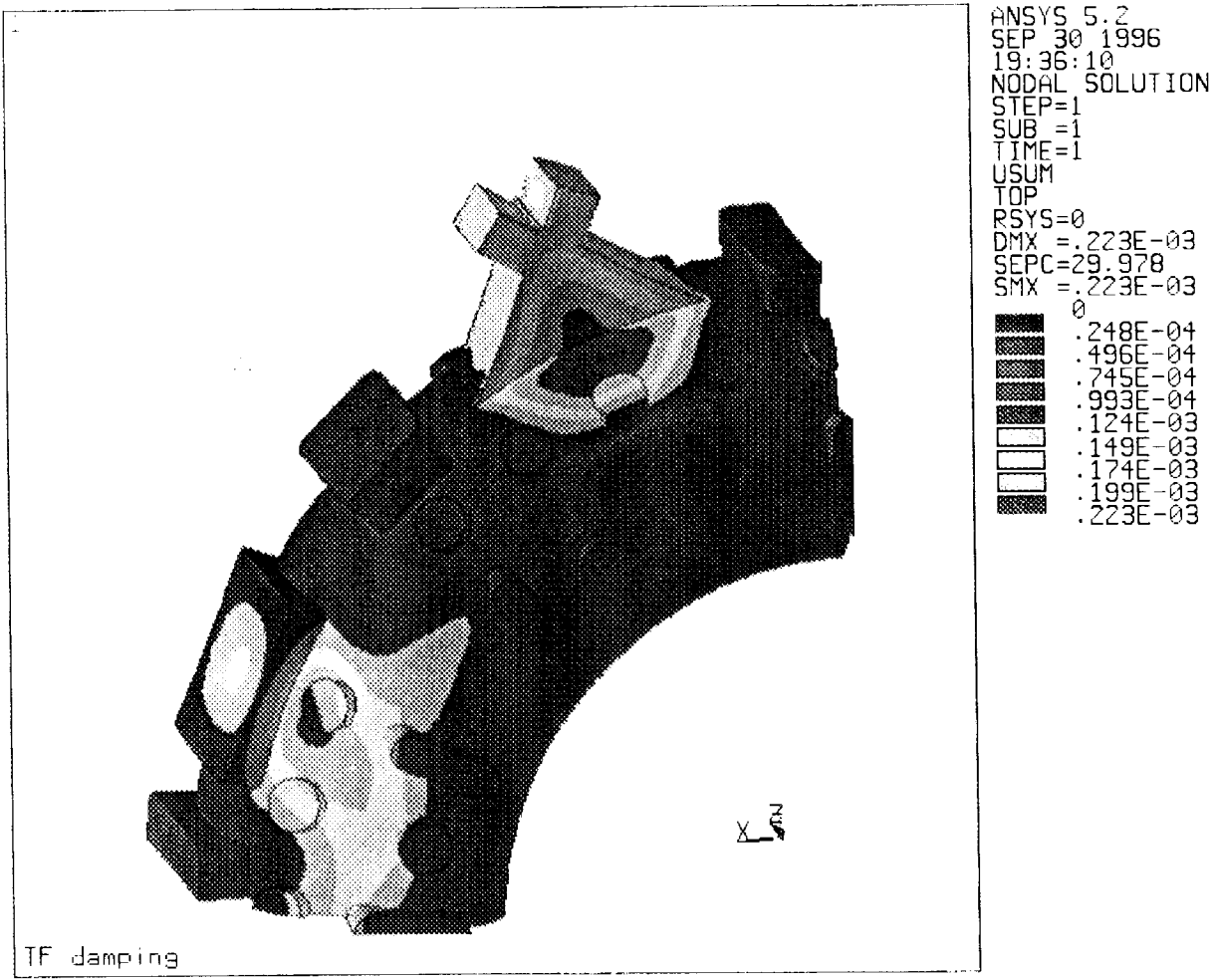


Fig. 18. Deformation of the vessel during TF damping.

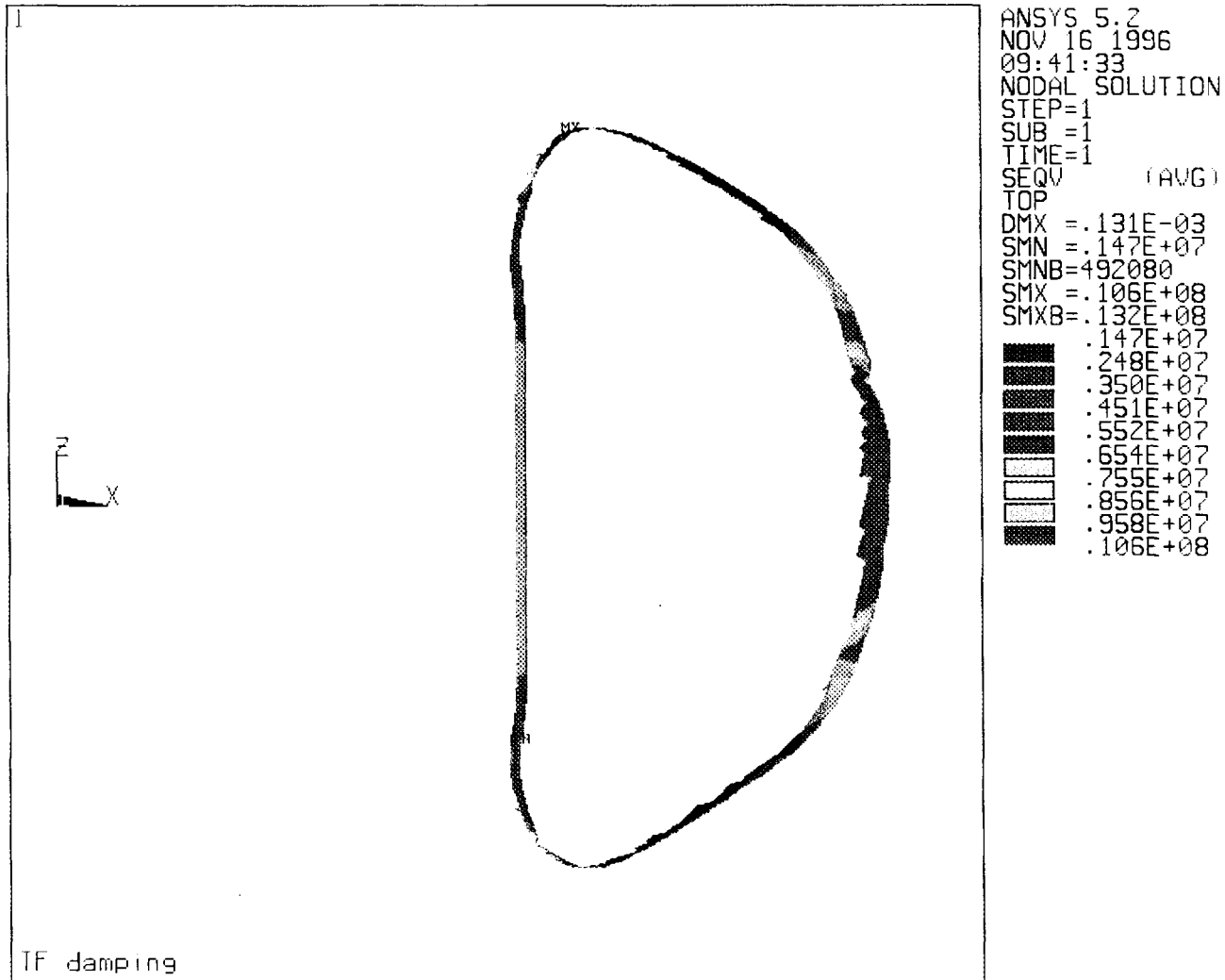


Fig. 19. Stress distribution on a poloidal belt section in TF damping.



Fig. 20. Stress distribution due to diamagnetism change during energy quench.

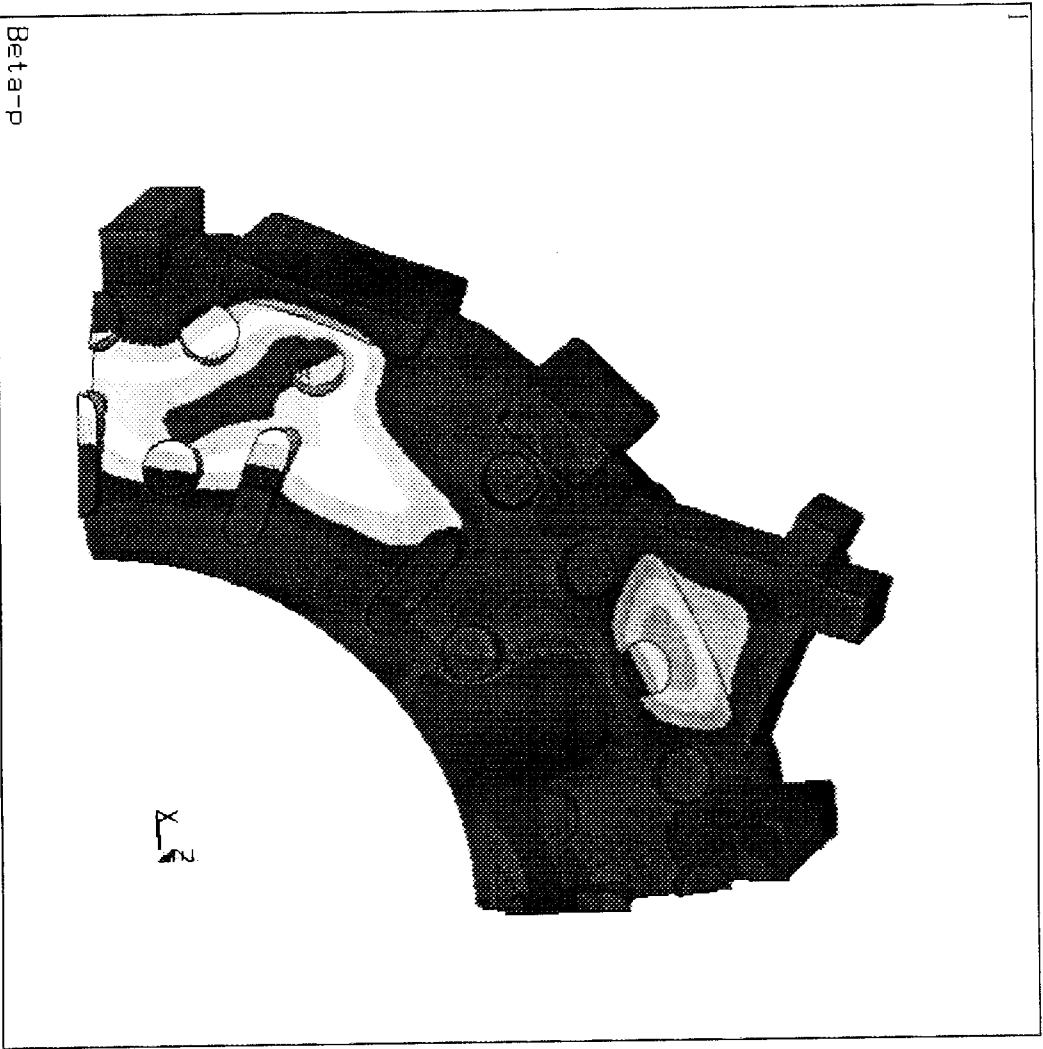


Fig. 21. Deformation of the vessel during energy quench.

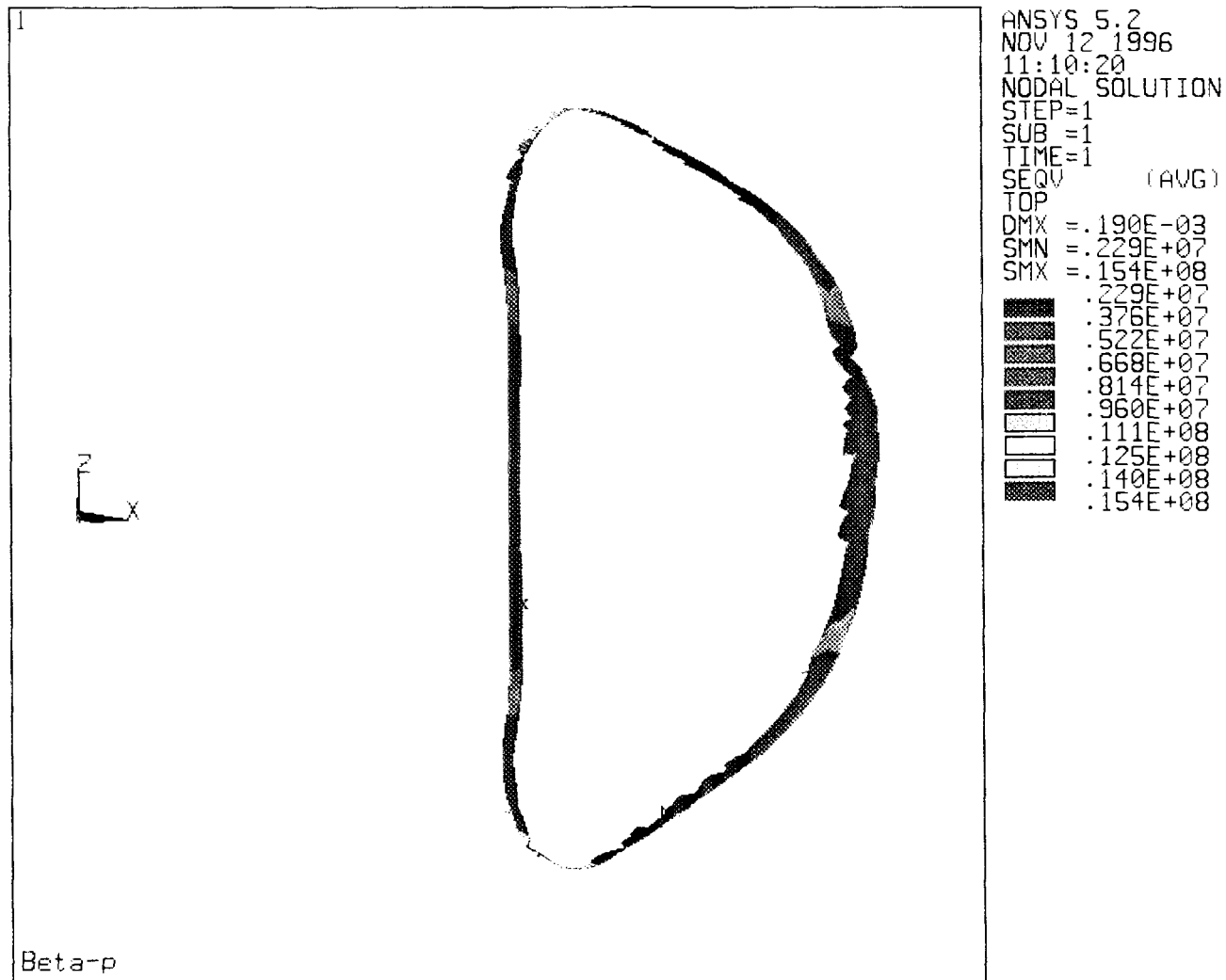


Fig. 22. Stress distribution on a poloidal belt section in the energy quench.

The maximum stress is  $0.680 \times 10^8$  Pa. This value is not so larger than expected considering that in the mode of TF damping, because the stress concentration is occurred around the NBI port (large major radius side) where the magnitude and resultant effect of induced pressures of  $1/r^2$  form are much lower than those of the atmospheric pressure. The maximum deformation is 0.273 mm.

Fig. 22 is the stress distribution on a poloidal belt section like as Fig. 19. As comparing the stresses on the linear part of the vessel ( $\theta \sim 180^\circ$ ) in the Figures 19 and 22, there is a large difference by  $\sim 2$  (equivalent to the ratio of the pressures exerted) between two values, which represents that the stress analysis has been performed properly.

## V. Conclusions

Analytic forms for the poloidal eddy currents flowing on the vessel, consequent pressures and forces acting on it were presented. The results were applied to typical operation modes of the KT-2 tokamak: machine start-up (TF coil charge), shut-down (discharge) and plasma disruption (diamagnetism change).

Stress analysis for two typical operation modes of TF damping in the machine shut-down and plasma energy quench during the plasma disruption were carried out using 3D FEM code (ANSYS 5.2). The results of the stress analysis are summarized as follows;

mode	Atmospheric pressure	TF damping	Energy Quench
Max. deformation	0.241 mm	0.223 mm	0.273 mm
Max. stress	$0.612 \times 10^8$ Pa	$0.558 \times 10^8$ Pa	$0.680 \times 10^8$ Pa
Most part of vessel	$0.680 \times 10^7 \sim 0.272 \times 10^8$ Pa	$0.620 \times 10^7 \sim 0.248 \times 10^8$ Pa	$0.756 \times 10^7 \sim 0.302 \times 10^8$ Pa
Linear part of vessel		$0.775 \times 10^7$ Pa	$0.154 \times 10^8$ Pa



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<p><u>초록 (300단어 내외)</u></p> <p>토카막 진공용기는 항상 대기압의 영향을 받는 것이외에 플라즈마 발생과 소멸까지 운전모드별로 각종의 전자기력을 받는다. TF 전자석 충전, TF 전자석 방전과 플라즈마 붕괴시의 energy quench의 모드에서 진공용기의 폴로이달 유기 전류가 발생한다. 용기에 흐르는 유기전류에 대한 일반적인 식과 그 전류로 인한 힘과 압력이 본 보고서에 표현되어 있습니다. 그 결과는 KT-2 토카막 운전시나리오에 적용된다.</p> <p>TF 전자석 방전과 플라즈마 붕괴시의 energy quench의 모드에서 생성되는 두 가지 전자기력에 관하여 3D FEM 코드인 ANSYS 5.2를 사용하여 진공용기의 응력해석을 수행하였습니다.</p>					
<p><u>주제명 키워드 (10단어 내외)</u></p> <p>eddy current, vacuum vessel, TF, KT-2, plasma, disruption, tokamak, scenario</p>					

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<u>Abstract (About 300 words)</u>					
<p>Poloidal eddy currents are induced on the continuous torus vacuum vessel by changes of the toroidal field during the machine start-up(toroidal field coil charge), shut-down(toroidal field coil discharge) and plasma disruption(plasma diamagnetism change). Analytic forms for the eddy currents flowing on the vessel, consequent pressures and forces acting on it are presented in this report. The results are applied to typical operation modes of the KT-2 tokamak.</p> <p>Stress analysis for two typical operation modes of toroidal field damping during a machine shut-down and plasma energy quench during a plasma disruption were carried out using 3D FEM code(ANSYS 5.2).</p>					
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