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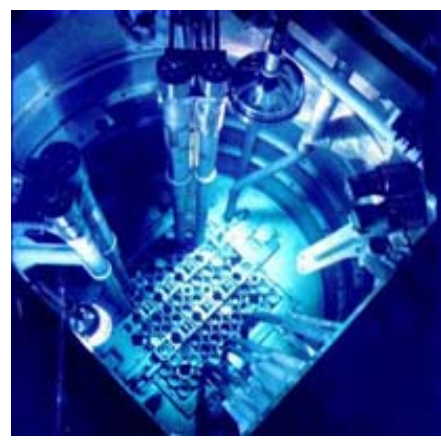
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The **Reduced Enrichment for Research and Test Reactors (RERTR) Program** was initiated by the United States Department of Energy in 1978 with the mission of developing the technologies necessary to convert research and test reactors from the use of fuels and targets containing highly-enriched uranium (HEU, = or > 20% U-235) to the use of fuels and targets containing low enriched uranium (LEU, < 20% U-235). This mission is consistent with the United States nonproliferation policy goal of minimizing and eventually eliminating the use of highly-enriched uranium in civil programs worldwide.

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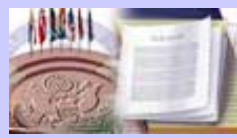


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**Abstracts and Papers Presented
at the 1999 International RERTR Meeting**

The 22nd International Meeting on Reduced Enrichment for Research and Test Reactors (RERTR) was held in Budapest, Hungary on October 3-8, 1999. The Abstracts and papers that were presented at this meeting are provided below.

National Programs

- [Progress of the RERTR Program in 1999](#)
A. Travelli
- [Moving into the 21st Century - The United States' Research Reactor Spent Nuclear Fuel Acceptance Program](#)
D.D. Huizenga, T.P. Mustin, E.C. Saris, J.E. Reilly
- [Status in 1999 of the High Flux Reactor Fuel Cycle](#)
J. Guidez, A. Gevers, G. Sordon, P.J.M. Thijssen, J.A. Hendriks, F.J. Wijtsma
- [The Russian RERTR Program Works Status](#)
P. Lavreniuk, V. Chernyshov, V. Aden, E. Kartashev, S. Bulkin, V. Likichev, A. Aleksandrov, A. Enin, V. Troianov, V. Popov, P. Egorenkov, V. Nasonov
- [Status of Reduced Enrichment Program for Research Reactors in Japan](#)
K. Kaieda, O. Baba, Y. Nagaoka, K. Kanda, Y. Nakagome
- [The French Development Program for a UMo Fuel](#)
R. Romano, J.L. Nigon, A. Languille, E. Le Borgne, H. Freslon

Fuel Development

- [Irradiation of Full Size UMo Plates](#)
H. Vacelet, P. Sacristan, A. Languille, Y. Lavastre, M. Grasse

- [Initial Assessment of Radiation Behavior of Very-High-Density Low-Enriched-Uranium Fuels](#)
G.L. Hofman, M.K. Meyer, J.L. Snelgrove, M.L. Dietz, R.V. Strain, K-H. Kim
- [Irradiation Behavior of Uranium-Molybdenum Dispersion Fuel: Fuel Performance Data from RERTR-1 and RERTR-2](#)
M.K. Meyer, G.L. Hofman, J.L. Snelgrove, C.R. Clark, S.L. Hayes, R.V. Strain, J-M. Park, K-H. Kim
- [Prototypic Irradiation Testing of High-Density U-Mo Alloy Dispersion Fuels](#)
S.L. Hayes, M.K. Meyer, C.R. Clark, J.R. Stuart, I.G. Prokofiev, T.C. Wiencek
- [Stable In-Reactor Performances at Low-Temperature of U-10wt.%Mo Dispersion Fuel Containing Centrifugally Atomized Powder](#)
K-H. Kim, J-M. Park, C-K. Kim, G.L. Hofman, M.K. Meyer, J.L. Snelgrove
- [An Investigation on \$\gamma\$ -U Phase Stability and Thermal Compatibility of Dispersion Fuel Meats Prepared with Atomized U-16at.%Mo, U-14at.%Mo-2at.%Ru, and U-14at.%Mo-2at.%Os](#)
C-K. Kim, K-H. Kim, Y-S. Lee, S-B. Park, G.L. Hofman
- [Fabrication and Characterization of Atomized U-Mo Powder Dispersed Fuel Compacts for the RERTR-3 Irradiation Test](#)
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- [BWX Technologies Progress with Reduced Enrichment Fuel Production](#)
B. Pace, E.A. Marinak
- [Some Tooling for Manufacturing Research Reactor Fuel Plates](#)
R.W. Knight
- [Melt-Dilute Treatment of Spent Nuclear Fuel Assemblies from Research and Test Reactors](#)
H.B. Peacock, T.M. Adams, A.J. Duncan, N.C. Iyer
- [Scrap Recovery Process for Oxide Fuel Plates](#)
B.G. Susanto, A. Muchsin, A. Sartono, A. Kadaryono, M.A. Akbar, B. Bull
- [High Density Uranium Silicide with Excess Uranium](#)
A. Suropto, S. Soentono, I.R. Subki, Prayoto, G.L. Hofman
- [Calculation of the Evolution of the Fuel Microstructure in UMo Alloys and Implications for](#)

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- [Progress on DART Code Optimization](#)

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- [Preparation Results for Lifetime Test of Conversion LEU Fuel in Plutonium Production Reactors](#)

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- [Beryllium Poisoning in the MARIA Reactor](#)

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- [A Level-Playing Field for Medical Isotope Production: How to Phase Out Reliance on HEU](#)
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