



### 6.3 The Effect of Helium Generation and Irradiation Temperature on Tritium Release from Neutron Irradiated Beryllium

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The effect of neutron irradiation condition on tritium release from beryllium is described in this paper. Beryllium samples were irradiated in the SM reactor with neutron fluence ( $E > 0.1$  MeV) of  $(0.37-2.0) \times 10^{22}$  cm<sup>-2</sup> at 70-100°C and 650-700°C. Mass-spectrometry technique was used in out of pile tritium release experiments during stepped-temperature anneal within a temperature range from 250 to 1300°C. The total amount of helium accumulated in irradiated beryllium samples varied from 521 appm to 3061 appm.

The first signs of tritium release were detected at temperature of 406-553°C. It was shown that irradiation temperature and helium generation level significantly affect the tritium release. A fraction of 44 - 74 % of tritium content in samples irradiated at low temperature (70-100°C) is released from beryllium at an annealing temperature below 800°C, whereas for samples after high temperature irradiation (650-700°C) tritium release did not exceed 14 %. Majority of tritium (~68 %) is released within a temperature range from 800 to 920°C. The increase of helium generation from 521 appm to 3061 appm results in lowering the temperature of maximal tritium release rate and the upper temperature of tritium release from beryllium by 100-130°C and 200-240°C, correspondingly. On the basis of data obtained, the diffusion coefficients of tritium in beryllium were calculated.

#### 1. INTRODUCTION

Knowledge of tritium release behavior in irradiated beryllium is an important aspect of beryllium application in future fusion devices. The tritium behavior in irradiated beryllium is a complex function dependent both on conditions of the irradiation (helium inventory, irradiation temperature, duration, etc.), and properties of beryllium (density, grain size, beryllium oxide contents, etc.). In spite of the fact that during recent years considerable progress was achieved in understanding of the above-stated problem [1-8], many aspects of tritium and helium behavior are poorly known. This paper presents the recent results of experiments on investigation of the effects of helium generation and irradiation temperature on tritium and helium release from beryllium.

#### 2. MATERIALS AND EXPERIMENTAL PROCEDURE

The study was performed for TE-56 and TshG-56 beryllium grades irradiated in high flux channels of the SM reactor. The irradiation parameters and some initial characteristics of beryllium are presented in Table 1 and Table 2.

Table 1  
Characteristics of beryllium

Grade of Be	Density, g/cm <sup>3</sup>	Grain size, μm	Be, wt. %	BeO, wt. %
TE-56	1.856	22-25	98.63	1.48
TshG-56	1.85	22-25	99.10	0.95

Tritium release kinetics was analyzed in a stepped isothermal annealing mode. Gases liberated under sample heating were collected in a closed volume equipped with omegatron mass-spectrometer (OMS). The OMS was calibrated against H<sub>2</sub> and <sup>4</sup>He standard leaks. Released amounts of <sup>4</sup>He were calculated from reading the OMS signal at mass 4. For evaluation of tritium released, only the mass 6

alone was used, since the fraction of tritium released in form of HT molecules was found to be negligible in our experimental conditions. A temperature was elevated by 20-170°C with every step of multi-stage annealing. A heating duration was between 0.6 and 4 hours, and the temperature ranged within 200-1300°C. At the final step of multi-stage annealing the samples were melted and the total amount of helium and tritium was measured.

Table 2  
Irradiation parameters of beryllium

Sample	Shape of sample	Neutron fluence, $E > 0.1 \text{ MeV, cm}^{-2}$	$T_{\text{irr,}} \text{ } ^\circ\text{C}$
#1 (TE-56)	Irregular	$0.5 \times 10^{22}$	70
#2 (TshG-56)	Disk d=6mm, h=0.5mm	$(0.37-0.59) \times 10^{22}$	650- 700
#3 (TE-56)	Disk d=6mm, h=0.5mm	$0.9 \times 10^{22}$	70
#4 (TE-56)	Irregular	$2 \times 10^{22}$	70- 100

### 3. EXPERIMENTAL RESULTS

#### 3.1. Tritium release

The results of the measurements on a fraction of released tritium as a function of temperature and duration of annealing are presented in Fig. 1; the tritium release rate is shown in Fig. 2.

The highest initial temperature of tritium release ( $T_s=553^\circ\text{C}$ ) was found for sample #1. When the temperature increased to 600°C the tritium release rate grew more than by a factor 30 and was constant up to 700°C. At  $T=760^\circ\text{C}$  the tritium release rate increased again by a factor 2.5 and then at 850°C increased additionally by a factor 2.2, approaching the maximum value ( $T_{\text{max rate}}$ ). At the following steps of annealing up to 1000°C, where tritium release was completed, the release rate decreased continuously, but remained rather high. Detritization of sample #1 completed at  $T = 1000^\circ\text{C}$  ( $T_{\text{upper}}$ ).

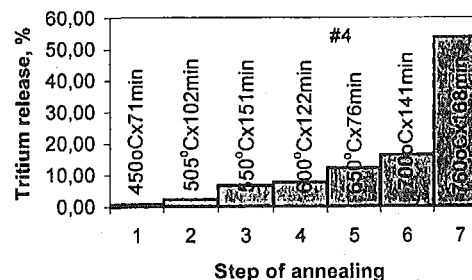
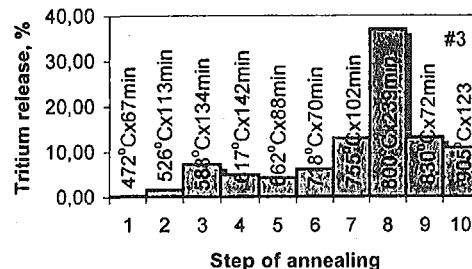
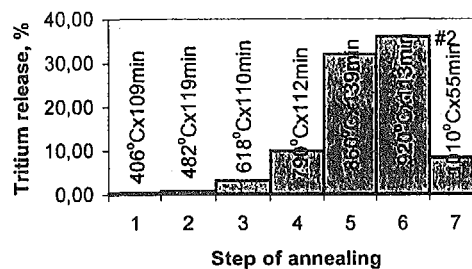
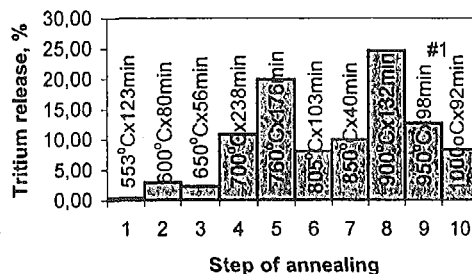


Fig.1. Tritium release (samples # 1-4)

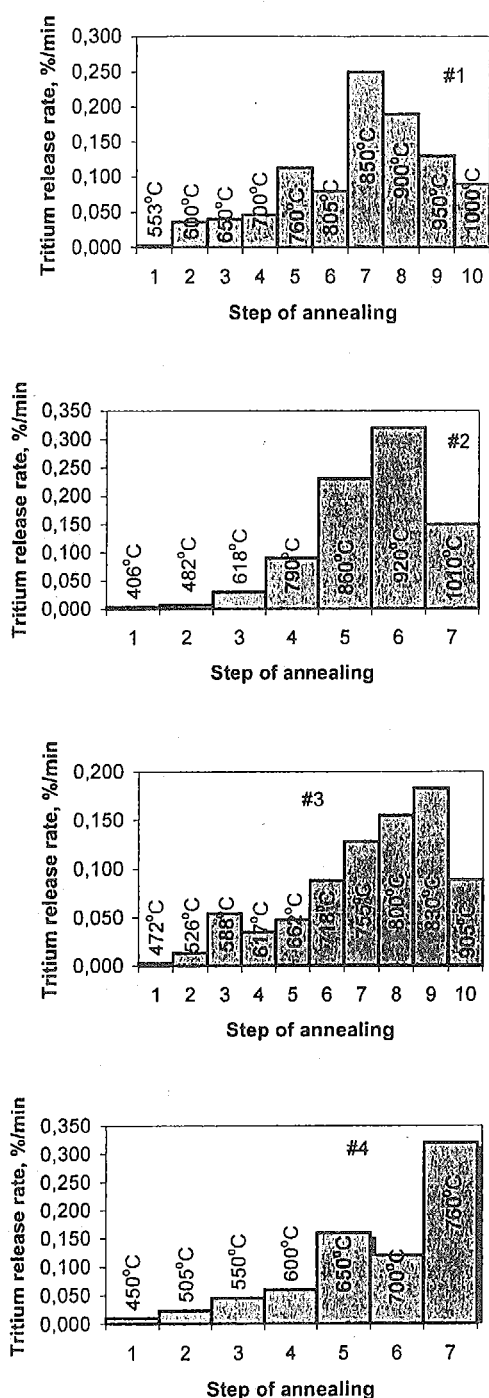


Fig. 2. Tritium release rate (samples # 1-4)

For sample #2 tritium release started at  $T_s=406^\circ\text{C}$ . At the subsequent steps of annealing the tritium release rate grew up more than by a factor 1.4-4.3 at each temperature step and reached the maximal value at  $920^\circ\text{C}$ . With further increase of temperature the tritium release rate fell down and detritization completed at  $T = 1080^\circ\text{C}$ . Majority of the tritium contained in the sample (~68 %) was allocated within a temperature range from  $800$  to  $920^\circ\text{C}$ . 90 % of the tritium was released when the temperature was increased to  $T = 1010^\circ\text{C}$ .

For sample #3 the tritium release started at  $T = 472^\circ\text{C}$ . At an annealing temperature of  $588^\circ\text{C}$  more than 10 % of the tritium inventory was released from the sample, and at an annealing temperature of  $800^\circ\text{C}$  about 37 % tritium was released from the sample after 4 hours of exposure. The integrated tritium release was thus 74 %. About ~90 % of the tritium release occurred at a temperature of  $830^\circ\text{C}$  and maximal tritium release rate corresponded to this temperature (Table 3). Detritization of sample #3 was completed at  $T = 905^\circ\text{C}$ .

For sample #4 an intensive tritium release was already observed at  $450^\circ\text{C}$ . At the subsequent steps of annealing the tritium release rate grew up by a factor 1.5 - 2.5 at each temperature step and reached a maximal value at  $760^\circ\text{C}$ , that coincided with the maximal tritium release rate from the sample ( $760^\circ\text{C}$ ). After three hours of exposure about 54 % of tritium contained in a sample was allocated from beryllium at  $T = 760^\circ\text{C}$ .

### 3.2. Diffusion of tritium

Using kinetic curves of gas release, the diffusion coefficients of tritium were calculated for samples #2 and #3 (Fig. 3) on the base of a solution of a differential equation for a thin plate. The results show that at temperatures below  $900^\circ\text{C}$  the diffusion mobility of tritium in sample #2 is lower than that in sample #3.

They also show, that at post irradiation annealing the retention of tritium in beryllium is higher for the samples irradiated at high-temperature ( $650-700^\circ\text{C}$ ) than for the samples, which were irradiated at low ( $50-100^\circ\text{C}$ ) temperatures.

Table 3  
Tritium Release parameters

Sample	Helium Content, appm	Tritium Content, appm	$T_s$ , °C	$T_{upper}$ , °C	$T_{max\ rate}$ , °C	Swelling, %
#1	521	6.5	553	1000	850	0
#2	601	12	406	1080	920	2
#3	1161	20	472	905	830	0.1
#4	3061	203	450	760	760	0.5-0.7

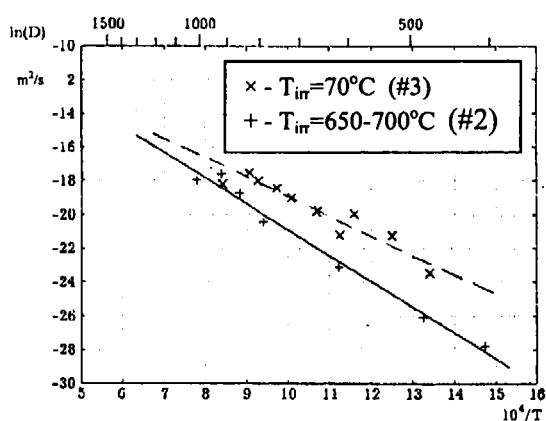


Fig. 3. Apparent tritium diffusion coefficients

#### 4. DISCUSSION

Among the factors, which can significantly affect the retention and release of tritium from beryllium, are the factors caused by operation conditions of beryllium in future fusion device. The most important from these factors are temperature and irradiation dose, and also their influence on the integrity of the beryllium structure (micro-cracks, swelling etc.) under the irradiation and at subsequent annealing.

The results of this research show, that at  $T < 400^\circ\text{C}$  in the of helium inventory range of 500 - 3100 appm tritium is completely captured in irradiated beryllium. The comparison of the samples irradiated at identical temperature conditions (#1, #3 and #4) shows that the increase of helium generation results in:

- Lowering the initial temperature of tritium release ( $T_s$ ) from  $553^\circ\text{C}$  to  $450^\circ\text{C}$ ;
- Lowering the upper temperature of tritium release ( $T_{upper}$ ) from  $1000^\circ\text{C}$  to  $760^\circ\text{C}$ ;
- Lowering the temperature of maximal rate of tritium release ( $T_{max\ rate}$ ) from  $850^\circ\text{C}$  to  $760^\circ\text{C}$ .

For the sample with the maximal helium generation (#4, 3061 appm) the temperature of the maximal rate of tritium release and the upper temperature of tritium release coincided with the initial temperature of helium release from a sample ( $760^\circ\text{C}$ ). For the other samples  $T_{upper} > T_{max\ rate}$ . This effect, apparently, is caused by distinction in the rate of structural evolution in beryllium (swelling, micro-cracks) during the irradiation and after irradiation annealing.

The results obtained confirm the assumption of study [7] that irradiation temperature significantly affects the tritium release and diffusion mobility of tritium. A fraction of 44 to 74 % of tritium in samples after low temperature irradiation ( $70\text{-}100^\circ\text{C}$ ) is released from beryllium at an annealing temperature below  $800^\circ\text{C}$ , while for samples after high temperature irradiation ( $650\text{-}700^\circ\text{C}$ ) tritium release did not exceed 14 %. Majority of tritium ( $\sim 68\%$ ) is released within a temperature range from 800 to  $920^\circ\text{C}$ . The irradiation temperature influences the upper temperature of tritium release similarly. For samples irradiated at low temperatures detrization completed at lower temperatures of annealing ( $\sim 905\text{-}1018^\circ\text{C}$ ) than for the sample irradiated at high-temperature irradiation ( $> 1050^\circ\text{C}$ ). These distinctions can be explained by the following reasons. In dense beryllium irradiated at low temperatures ( $70\text{-}100^\circ\text{C}$ ) practically all tritium formed during the irradiation remains in beryllium,

while during the high-temperature irradiation part of the tritium diffuses out from beryllium. It occurs due to higher diffusion mobility of tritium at high temperatures, and also as a result of an imperfection of structure integrity because of swelling and the formation of helium porosity (Table 3). With higher temperature of irradiation and swelling of beryllium, bigger amount of tritium diffuses out from beryllium during the irradiation [7]. However, tritium remaining in beryllium after the high-temperature irradiation apparently is in a more stable energy condition than after low temperature irradiation [7]. Therefore, a relatively small fraction of tritium (<15 %) is released from beryllium irradiated at high temperature when the temperature of post- irradiation annealing  $T_{ann} < T_{irr}$ , while the remaining fraction will be released at  $T_{ann} > T_{irr}$ . With higher irradiation temperature and lesser swelling, bigger amount of tritium is retained in beryllium at post irradiation thermal expose.

## 5. CONCLUSION

The level of helium generation and irradiation temperature significantly affect the kinetics of tritium release from beryllium at stepped isothermal annealing. The main results are as follows:

1. The increase of helium generation from 521 appm to 3061 appm results in:

- Lowering the initial temperature of tritium release by  $\sim 100^\circ\text{C}$  ( $T_s$  reduced from  $553^\circ\text{C}$  to  $450^\circ\text{C}$ );
- Lowering the upper temperature of tritium release by  $\sim 240^\circ\text{C}$  ( $T_{upper}$  reduced from  $1000^\circ\text{C}$  to  $760^\circ\text{C}$ );
- Lowering the temperature of maximal tritium release rate by  $\sim 90^\circ\text{C}$  (reduced from  $850^\circ\text{C}$  to  $760^\circ\text{C}$ ).

2. Irradiation temperature significantly affects the tritium release and diffusion mobility of tritium. A fraction of 44 to 74 % of tritium in samples after low temperature irradiation ( $70\text{-}100^\circ\text{C}$ ) is released from beryllium at an annealing temperature below  $800^\circ\text{C}$ , while for samples after high temperature irradiation ( $650\text{-}700^\circ\text{C}$ ) tritium release did not exceed 14 %. Majority of tritium ( $\sim 68$  %) is released within a temperature range from  $800$  to  $920^\circ\text{C}$ . Detritization of samples after low

temperature irradiation completed in a temperature range from  $\sim 905^\circ\text{C}$  to  $1018^\circ\text{C}$ , and that for the sample after high-temperature irradiation occurred at a temperature of  $1080^\circ\text{C}$ .

3. The apparent diffusion coefficients of helium and tritium were calculated for a temperature range from  $400^\circ\text{C}$  to  $1000^\circ\text{C}$ . At temperatures below  $900^\circ\text{C}$  the diffusion mobility of tritium for the sample irradiated at high temperatures ( $650\text{-}700^\circ\text{C}$ ) is lower than for the samples irradiated at low temperatures ( $50\text{-}100^\circ\text{C}$ ).

## REFERENCES

1. D.L. Baldwin and M.C. Billone, Diffusion/Desorption of Tritium from Irradiated Beryllium, *J. Nucl. Mater.* 212-215 (1994) 948-953.
2. E. Ishitsuka, H. Kawamura and T. Terai, Effect of Oxide Layer on Tritium Release from Beryllium Pebbles, Proc. of 18<sup>th</sup> SOFT, Fus. Tech., vol. 2, 1994, p. 1345-1348.
3. F. Scaffidi-Argentina and H. Werle, Tritium Release from Neutron Irradiated Beryllium: Kinetics, Long-Time Annealing and Effect Crack Formation, Proc. 2nd IEA Int. Workshop on Beryllium Technology for Fusion, Jackson Lake Lodge, Wyoming, USA, Sept. 6-8, 1995, p. 235-248.
4. R.A. Anderl, G.R. Longhurst, M.A. Oates and R.J. Pawelko, Tritium and Helium Retention and Release from Irradiated Beryllium, Proc. Third IEA Int. Workshop on Beryllium Technology for Fusion, Oct. 22-24, 1997, Mito, Japan, JAERI-Conf 98-001, p. 307-315.
5. M. Dalle-Donne, D.L. Baldwin, D.S. Gelles, L.R. Greenwood et al., Behavior of Beryllium Pebbles under Irradiation, Proc. Third IEA Int. Workshop on Beryllium Technology for Fusion, Oct. 22-24, 1997, Mito, Japan, JAERI-Conf 98-001, p. 296-306.
6. R. Rolli, S. Rubel, H. Werle, C.H. Wu, Influence of Neutron Irradiation on the Tritium Retention in Beryllium, Third IEA Int. Workshop on Beryllium Technology

- for Fusion, Oct. 22-24, 1997, Mito, Japan, JAERI-Conf 98-001, p. 228-233.
7. I.B. Kupriyanov, V.V. Vlasov, Helium and Tritium Behavior in Neutron Irradiation Beryllium, Proc. 4<sup>th</sup> Int. Workshop on Beryllium Technology for Fusion, Sept. 14-16, 1999, Karlsruhe, Germany, p. 264-271.
  8. I.B. Kupriyanov, V.A. Gorokhov, V.V. Vlasov, A.M. Kovalev, V.P. Chakin, The Effect of Irradiation Dose on Tritium and Helium Release from Neutron Irradiated Beryllium, Proc. 5<sup>th</sup> Int. Workshop on Beryllium Technology. For Fusion, Oct. 10-12, 2001, Moscow, Russia, will be published.